

ENEA-INMRI



Iride: The particle Source

L. Quintieri

INMRI : ENEA

ENEA-Istituto Nazionale di Metrologia delle Radiazioni Ionizzanti - Casaccia

Frascati, LNF

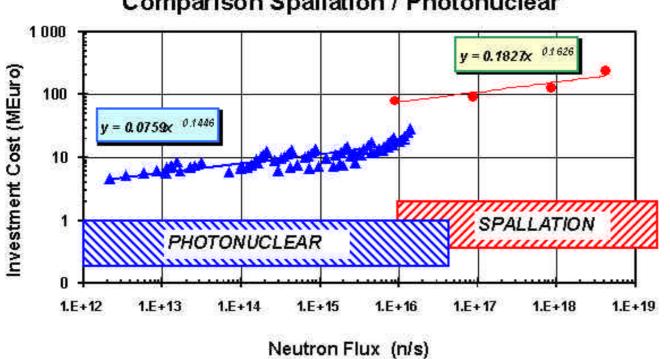
June 10-11, 2013

Outline

- The Starting Point: Measurements of photo-produced neutrons at Daφne: the n@BTF Project. ---> "quickly" described
- Some brief considerations on spallation vs photo-production accelerator driven systems
- IRIDE: particle yield and energy spectra
 - Source term: study and design of an optimised target for neutron production
 - Preliminary MC estimations of the particle yields (not only neutrons: p, pi, muons, etc)
 - Expected Source Strength comparison with the corresponding ones of other worldwide accelerator driven facilities
 - Description of the thermo-mechanical issues: an analytical approach and need of FE method simulations for the thermo-mechanical issues

Spallation vs Photoneutron sources

- Spallation sources are very effective in producing neutrons. Example: 100 MeV e- on U target--> 0.11 n/e-, so 900 MeV are needed to produce 1n. On the contrary, on the same target 1GeV p --> 30 n (a factor 30 more effective)... but they are large and expensive.
- Electron drivers although are much less effective in neutron production are rather cheap and compact machine that might also bring advantages in terms of reliability



Comparison Spallation / Photonuclear

To obtain very high neutron fluxes by photo-production, much higher electron beam intensity will be necessary. This will increase the electron accelerator complexity, resulting in a less convenient solution from an engineering and economical point of view.

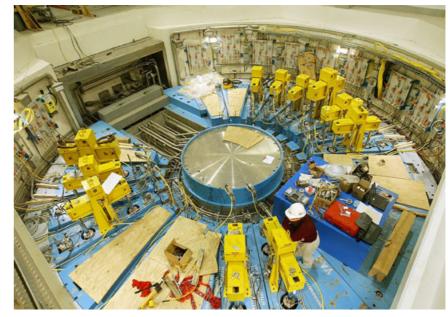
Above a given neutron rate (10¹⁶ n/s) the spallation will be preferred while for the lower fluxes, the photonuclear process will tend to be more convenient

Plot reference:

Ref: D. Ridikas, H.Safa, M.L.Giacri – Conceptual Study of Neutron Irradiator Driven By Accelerator – 7th Information Exchange Meeting on Actinide and Fission Product P&T (NEA/OCDE), Jeju, Korea, 14-16 Oct. 2002

Neutron Sources around the world

- SNS (Spallation Neutron Source) at Oak Ridge
- ISIS (Spallation Neutron Source) in England
- n-TOF (Spallation Neutron Source) at CERN
- IPNS (Intense Pulsed Neutron Source) at ANL
- LANSCE (Spallation Neutron Source) Los Alamos
 Neutron Source
- Gelina (Geel Electron Linear Acceleration) in Geel Belgium (Photo-production)
- n-ELBE (Photo-production) at Rossendorf, Germany
- ORELA at Oak Ridge (Photo-production)
- ESS (European Spallation Source) at Lunde Sweder



SNS at Oak Ridge Electron Beam Neutron Sources

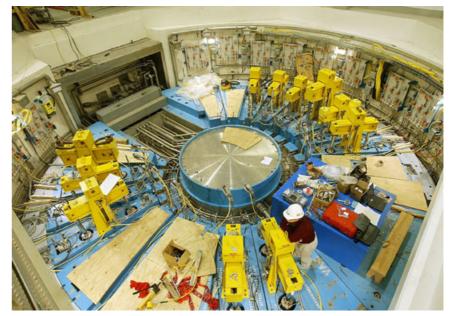
Conventional Linac

	Nation	Institute	Beam Energy	Target
GELINA	Belgium	IRMM	105 MeV (Avg.)	U
ORELA	USA	ORNL	180 MeV	Та
Gaerttner Linac	USA	RPI	60 MeV	Та
KURRI Linac	Japan	Kyoto U.	46 MeV	Та
IREN	Russia	JINR	200 MeV	U
PNF	Korea	POSTECH	100 MeV	Та

- SC Linac
 - : nELBE (FZD, Germany, 40 MeV, Pb liquid target)

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Iride SC Linac				

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OPTIMISED TARGET & EXPERIMENTAL APPARATUS

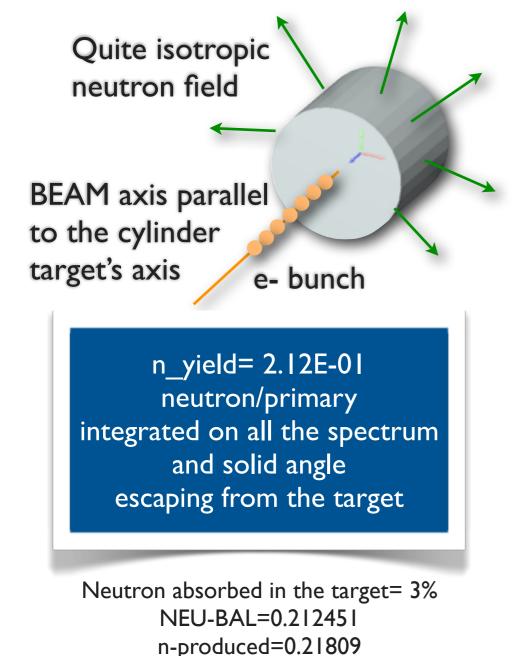
Acquired know-how and MC validations

The Optimized Target

Optimization criterion: recursive process on calculating neutron fluence leaving the target, increasing linear dimensions (Rand L) Best solution: the one for which, a new step would have affected only marginally the photoneutron yield (gain of only few %). L from 15 to 20X0 gain less than 3% (so final choice 17 X0 on the plateau); Same considerations: R final choice 10 X0.

W cylinder R=35 mm L =60

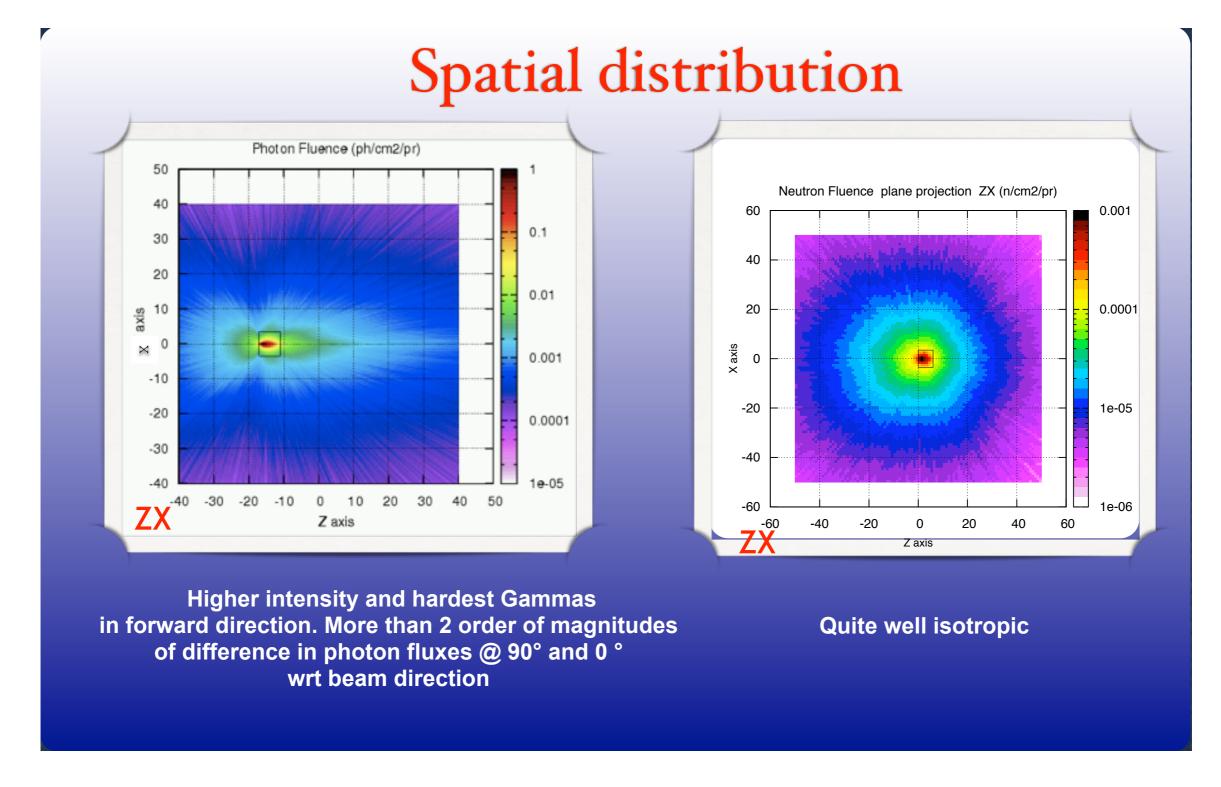
mm(Z=74; e=19 g/cm3; X0=0.35 cm; MR=0.9





Maximum rate @ 0.7 MeV= 0.1343 n/primary integrated on all the solid angle

Since the maximum electron beam spot size on the target is enclosed in a circle of I.cm radius and the accuracy by which we can set the beam position is better of few mm, we can be confident that all the energy of the primary electrons will be ever deposited in the target



The photon and neutron fluxes have been calculated by the USRYIELD card for each direction

Neutron and Photon Flux (Target and air around).

(Calculation with 25000 primaries)

Angle wrt beam direction	Photons[ph/cm2/pr] @0.5 m	Neutron [n/cm2/pr] @0.5 m
0°	1.16559E-02 +/- 1.207616 %	5.78188E-06 +/- 0.5680834 %
-30°	3.18765E-04 +/- 2.074163 %	7.32548E-06 +/- 1.397449 %
30°	2.00091E-03 +/- 0.6900502 %	6.98712E-06 +/- 0.2340965 %
-45°	2.55639E-04 +/- 1.500333 %	6.73067E-06 +/- 1.536476 %
45	9.85524E-04 +/- 0.7157903 %	6.37311E-06 +/- 0.8208705 %
-60	1.80074 E-04 +/- 3.305821 %	5.84105E-06 +/- 1.092179 %
60°	4.76631E-04 +/- 1.785744 %	5.35342E-06 +/- 1.501851 %
90°	9.61925E-05 +/- 4.184312 %	4.37955E-06 +/- 1.058816 %

Photon and Neutron Flux integrated on all the solid angle. They are inversely proportional to the square of distance

	Photons[ph/cm2/pr]		Neutron [n/cm2/pr]	
@ 0.5 m	6.2910217E-04 +/- 0.3605311	%	5.8066257E-06 +/- 0.5866572	%
@ I.0 m	.5729200E-04 +/- 0.3733959	%	1.4539921E-06 +/- 0.5513448	%

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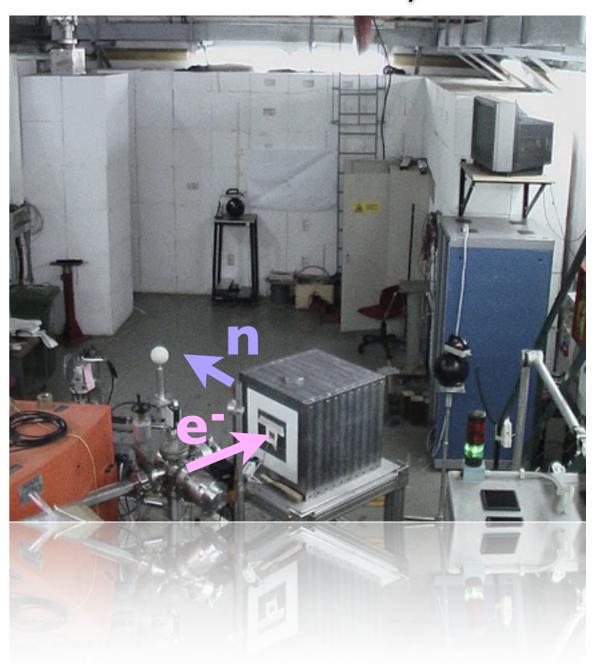
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Final Experimental SET-UP



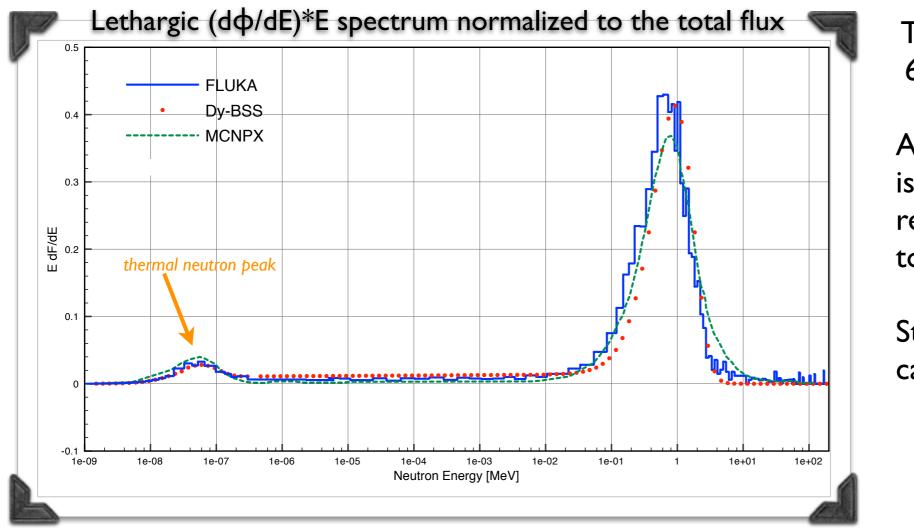
First Measurements May 2010



2 environmental detectors: response from thermal up to 20 MeV. The constance of ratio of their readings assure to have done measurements in a satisfactory repeatability or irradiation condition within $\pm 1.5\%$

Experimental and Computational neutron Spectra

The point of test was at ~150 cm from the target and at 90° wr to the impinging electron beam line



Total Neutr	on Flux per prir	nary particle

8.04E-7 ±3%	8.10E-7 ±4%	8.02E-07 ±0.2%
Ex. Measurement	FLUKA	MCNPX

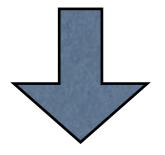
This make us confident in being reliable in source term estimation by MC

The flux above 10 keV is 6.53E-7 /cm2/pr

As expected, more than 80% is found around the Giant resonance (from 10 KeV up to 20 MeV)

Statistical uncertainty in the calculations less than 4%

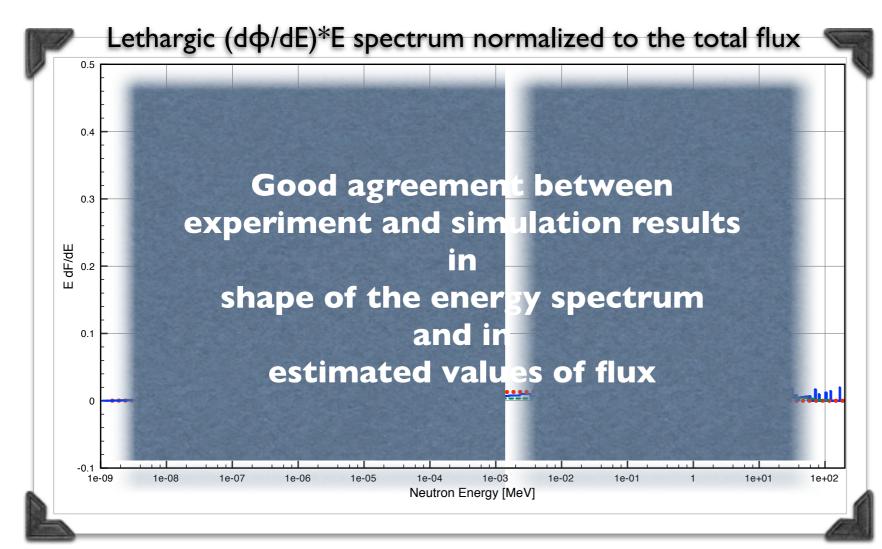
Max neutron Flux currently available in BTF



Neutron Flux at 1.5m from shield = 4E+5 n/cm2/s corresponds to Equivalent Dose=45 mSv/h

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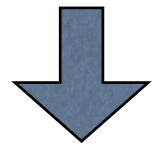
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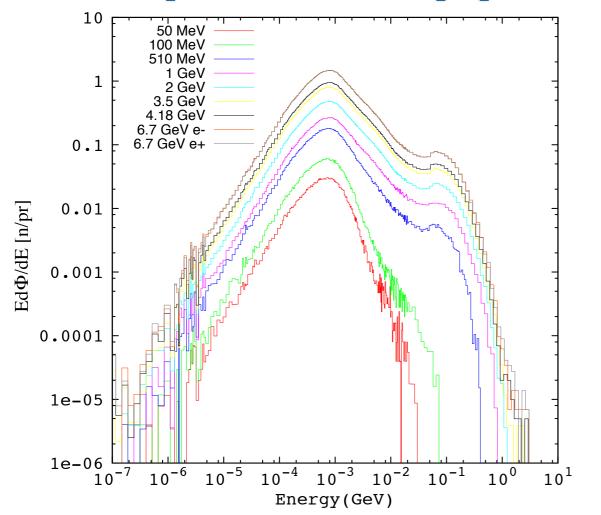


IRIDE

THE NEUTRON SOURCE

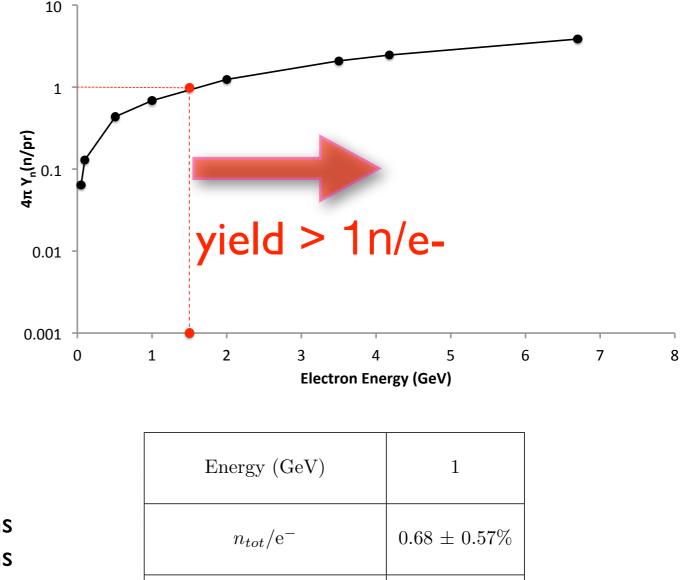
PRIMARY ELECTRON ENERGY RANGE: 1-3 GEV

Neutron source vs primary electron energy: some preliminary predictions for the W case



Increasing the energy of the primary electrons above 500 MeV, the contribution of the neutrons produced in the upper part of the continuous spectrum becomes no more negligible, even if it is still much lower respect to the one coming from the Giant Dipole Resonance.

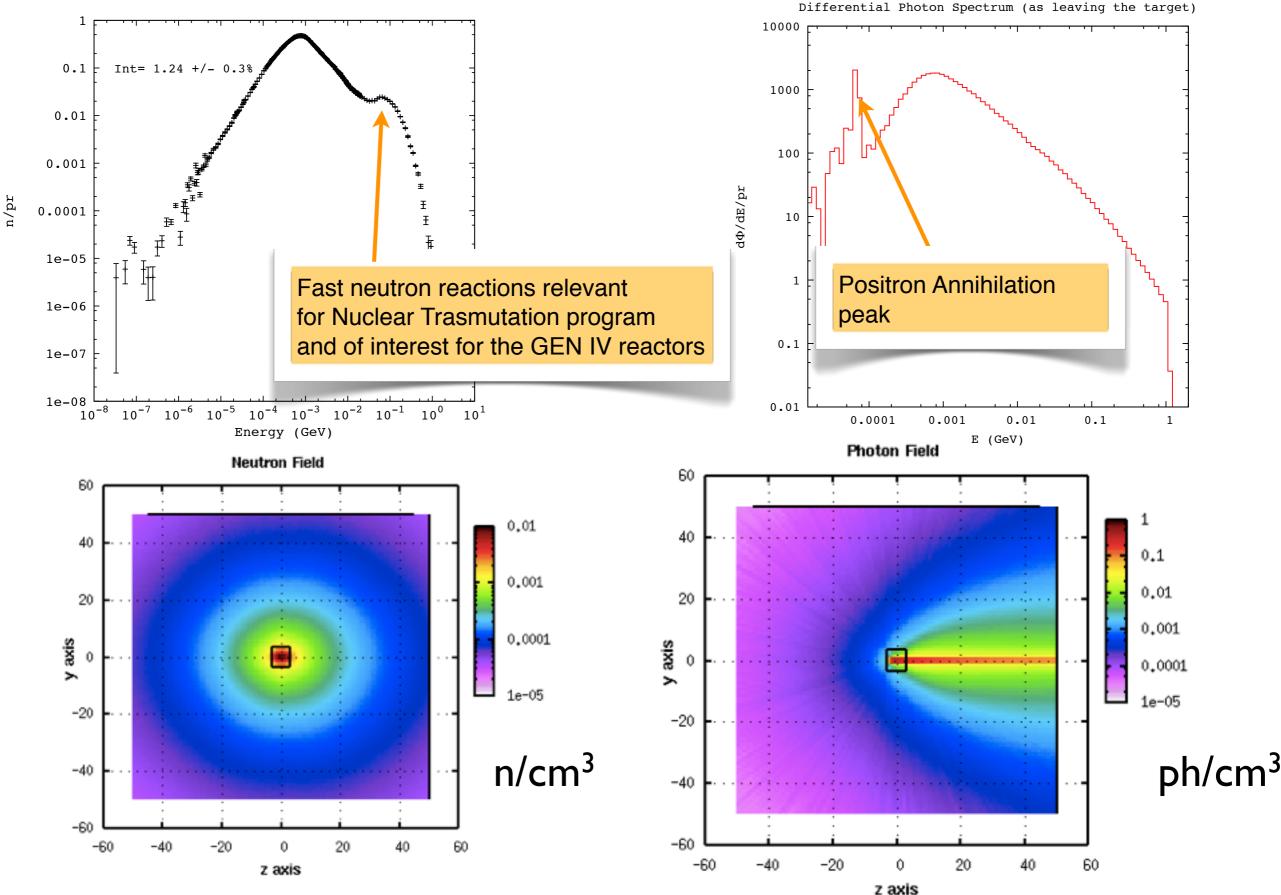
We have estimated that the percentage of neutrons above 20 MeV is a factor 4 higher for I GeV electrons than for 100 MeV electrons.



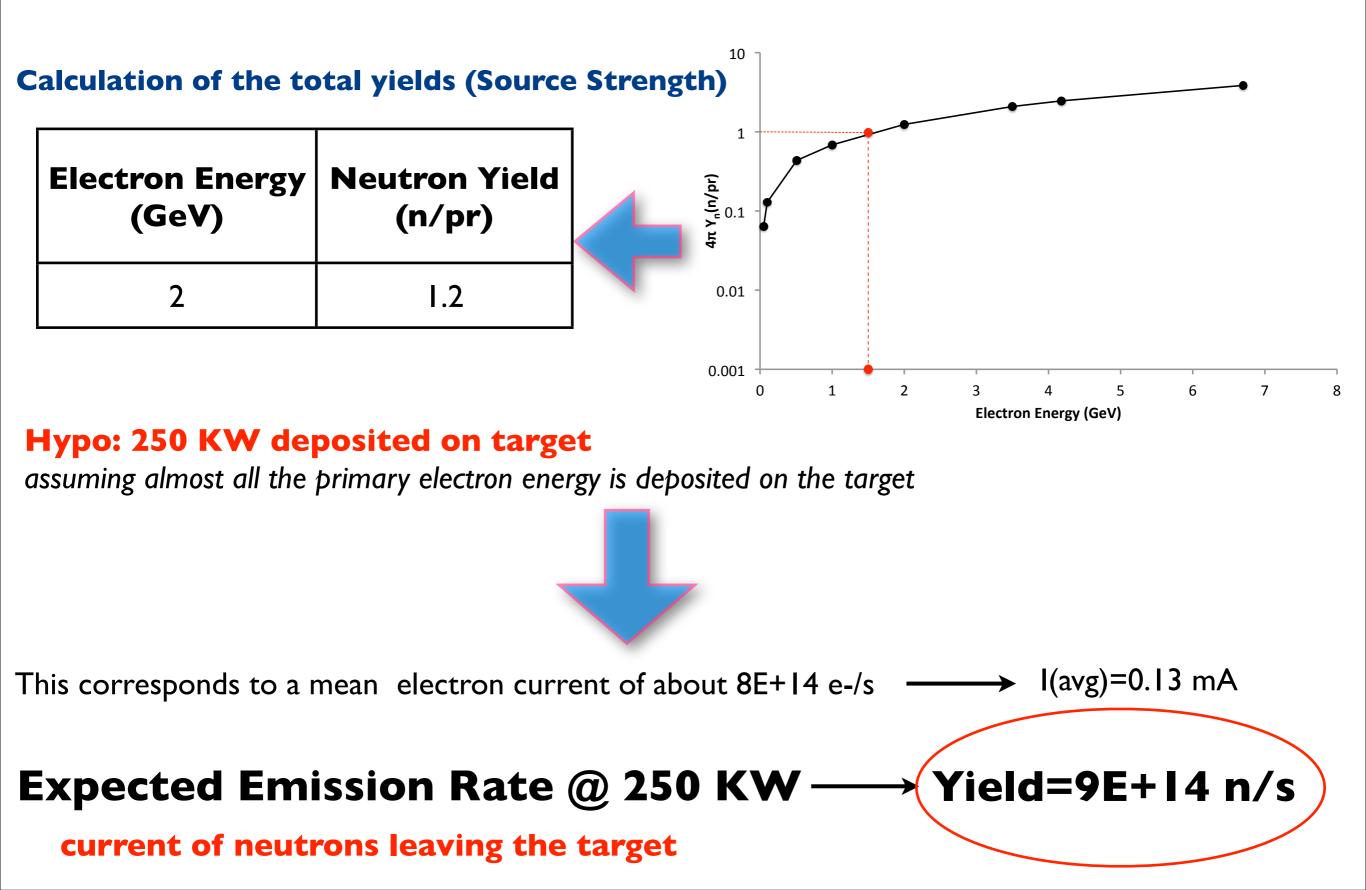
$\frac{1}{n_{tot}} \int_{10MeV}^{\infty} \Phi(E) \mathrm{d}E \left[\%\right]$	6
$\frac{1}{n_{tot}} \int_{20MeV}^{\infty} \Phi(E) \mathrm{d}E \left[\%\right]$	4
$\frac{1}{n_{tot}} \int_{100MeV}^{\infty} \Phi(E) \mathrm{d}E \left[\%\right]$	1

Fluka Estimation accuracy < 1%

Neutrons and Gammas produced in the target for 2 GeV electrons



IRIDE neutron source: expected neutron rate for the opt. W target and 2GeV e-



W target: the expected emission rates vs the deposited power

Deposited Power (kW)	Primary Electron Energy (GeV)	Expected Average Emission Rate
30	I	I.30E+I4
250	I	I.00E+I5
400		I.70E+15
30	3	4.30E+13
250	3	3.30E+14
400	3	5.60E+14

At energies above I GeV and for a given thick target (i.e, able to enclose the whole electromagnetic cascade), **the neutron production rate depends exclusively on the electron rate**, i.e. to the ratio between power and electron energy, This because the yield per primary goes toward saturation



IRIDE VERSUS OTHER FACILITIES:

"QUANTITATIVE" COMPARISON OF THE SOURCE STRENGTH

AND

"QUALITATIVE" COMPARISON FOR THE ENERGY SPECTRUM

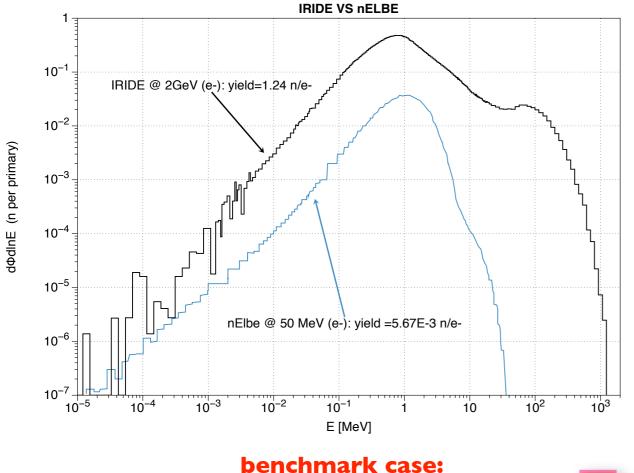
Source Strength Comparison: Iride vs some other accelerator driven neutron facilities

Facility Parameters	nElbe*	Gelina**	nToF	ISIS	IRIDE
Source	SC e- Linac	e-Linac	p spallation	p spallation	SCe- Linac
Part E (MeV)	40	120	20000	800	2000
Max Power (kW)	18	11	45	160	250 (30)
Neutron/s ***	3.40E+13	3.20E+13	8.10E+14	1.00E+16	9.4E+14(1.1E+14)
Flight Path (m)	4	8-400	185	60	?
Pulse width (n)	<0.4	I-2000	7	300	?
Rep Rate (Hz)		up to 900	0.278-0.42		?
Best Intrinsic Resolution (ns/m)		0.0025	0.034		?

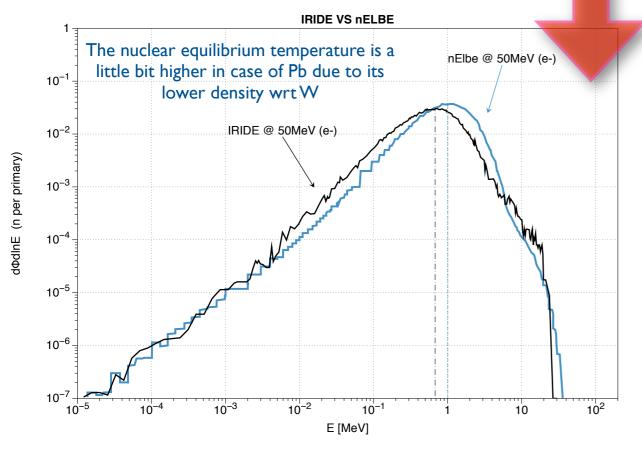
** see next talk of Anna Ferrari

** talk given by Jan Heyse

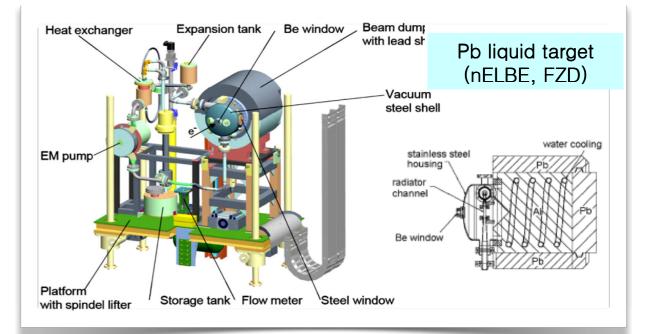
Iride vs nElbe: source term comparison



IRIDE and nELBE with e- @50 MeV--



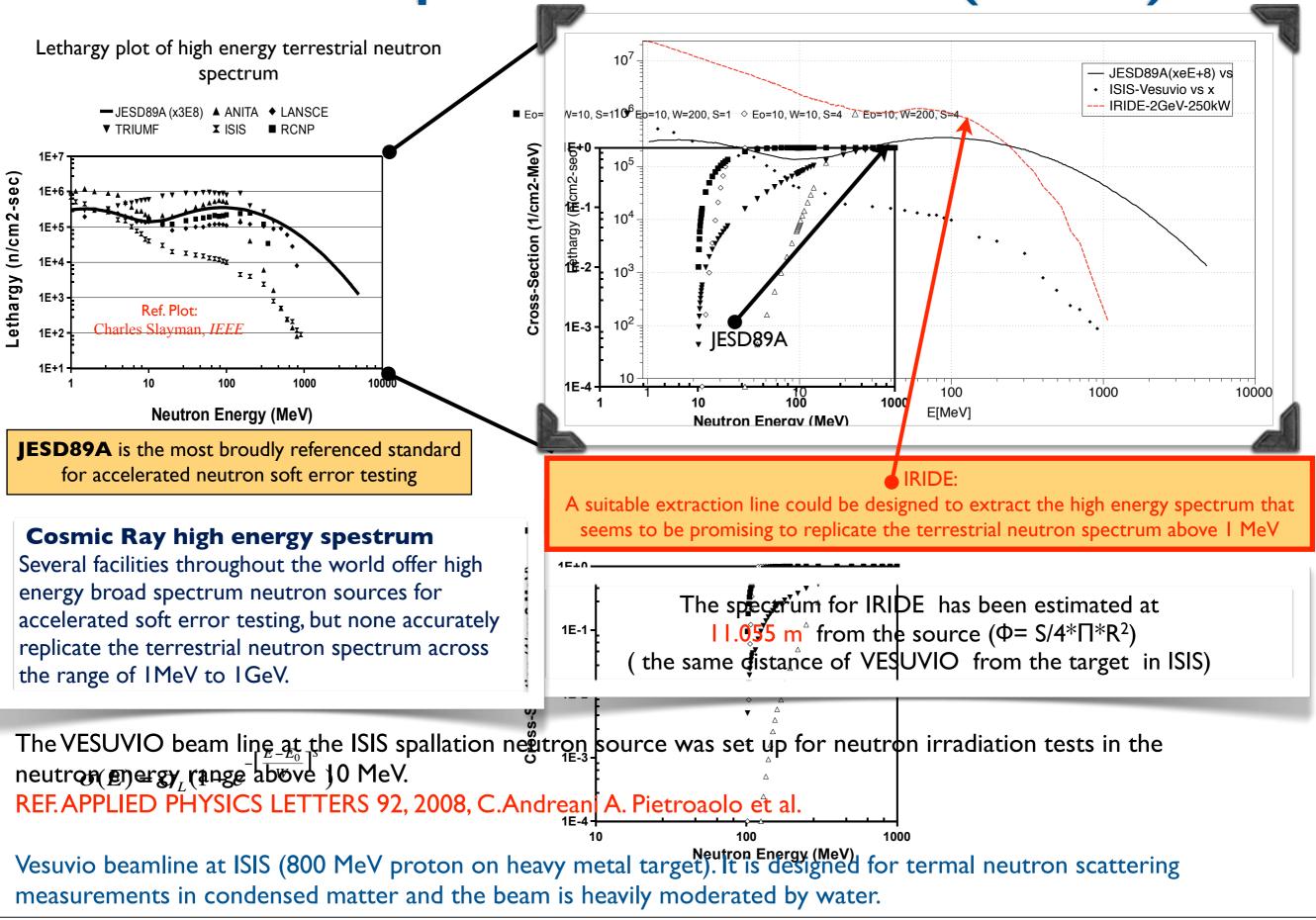
nELBE: SC Linac; 30-40 MeV electrons on liquid Pb Avg Current: I mA (18 kW)

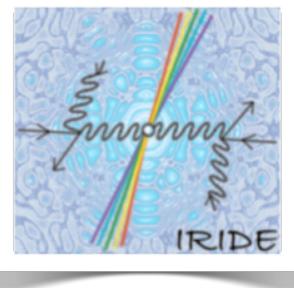


Neutron Yield [n/e-]

FACILITY	Ee-(MeV)	Yield (n/e-)	
Iride	2000	1.2	
nElbe	40	5.67E-03	

Iride and terrestrial Cosmic Ray Spectrum: indirect comparison with ISIS source (Vesuvio)





IRIDE: NOT ONLY NEUTRONS

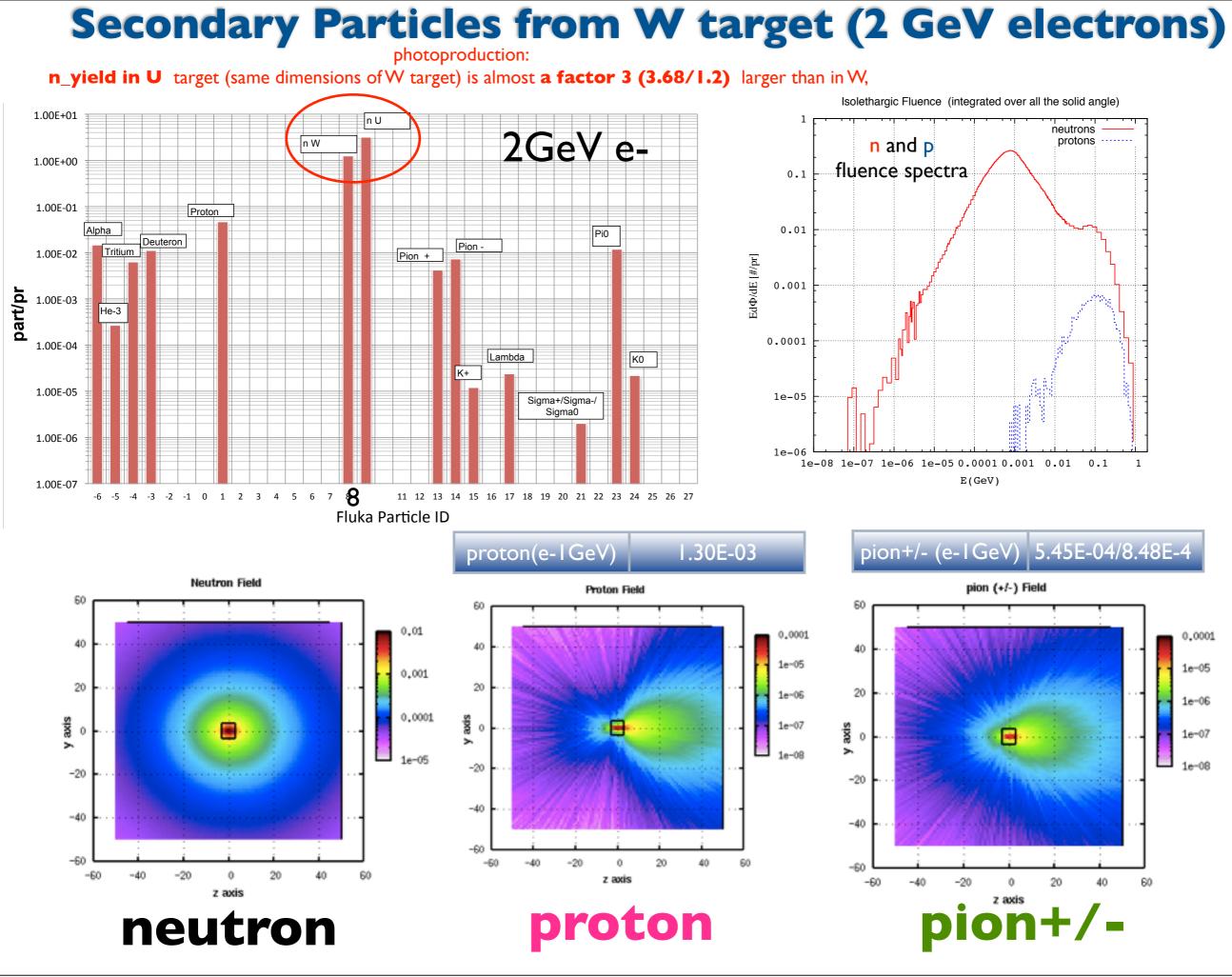
The optimized target could be composed of different nested materials, depending on the particles we are interested in extracting: this means a "high Z" core and a "medium Z" external blanket to lower the coloumbian barrier for charged particles

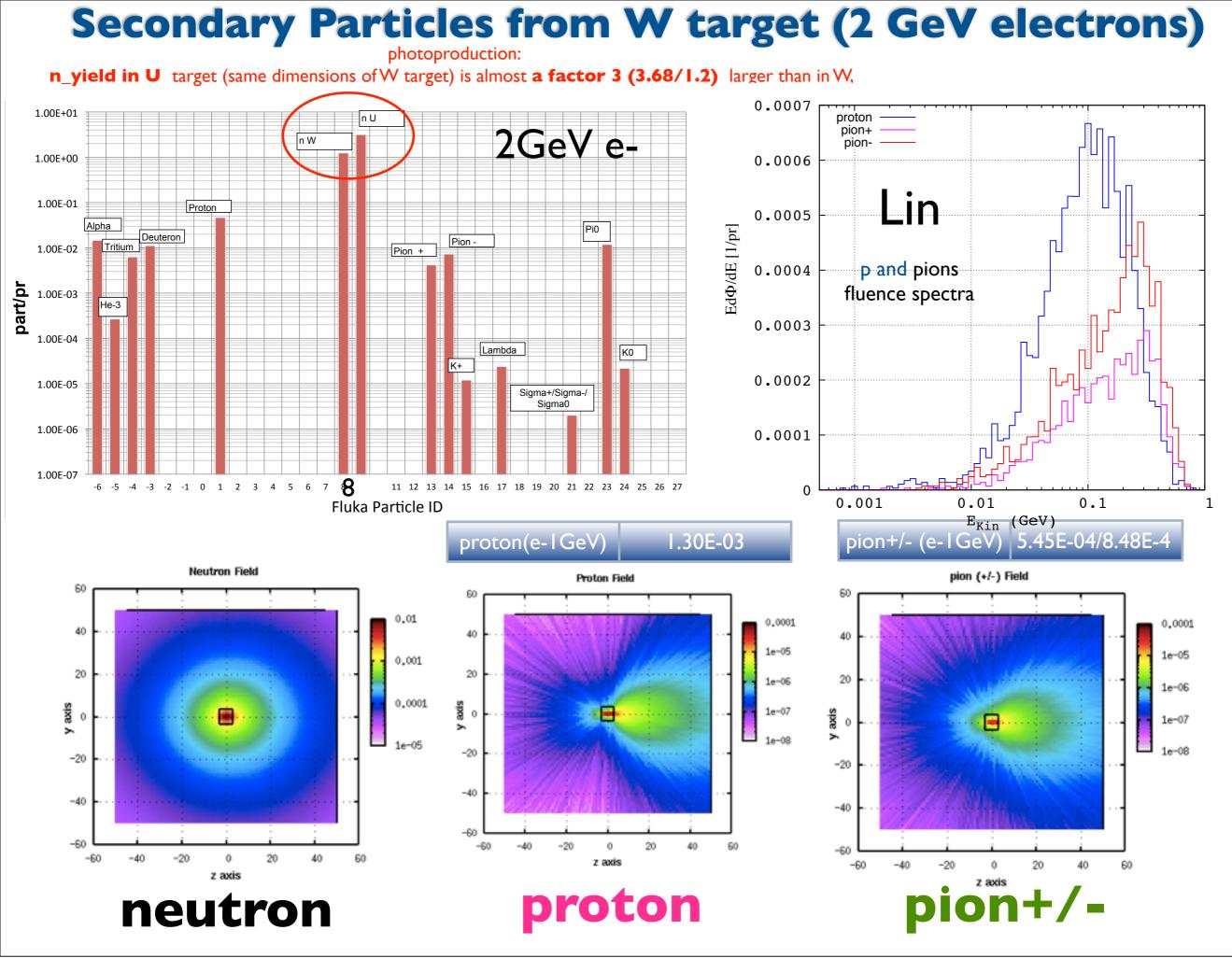
Emanuele Ripiccini

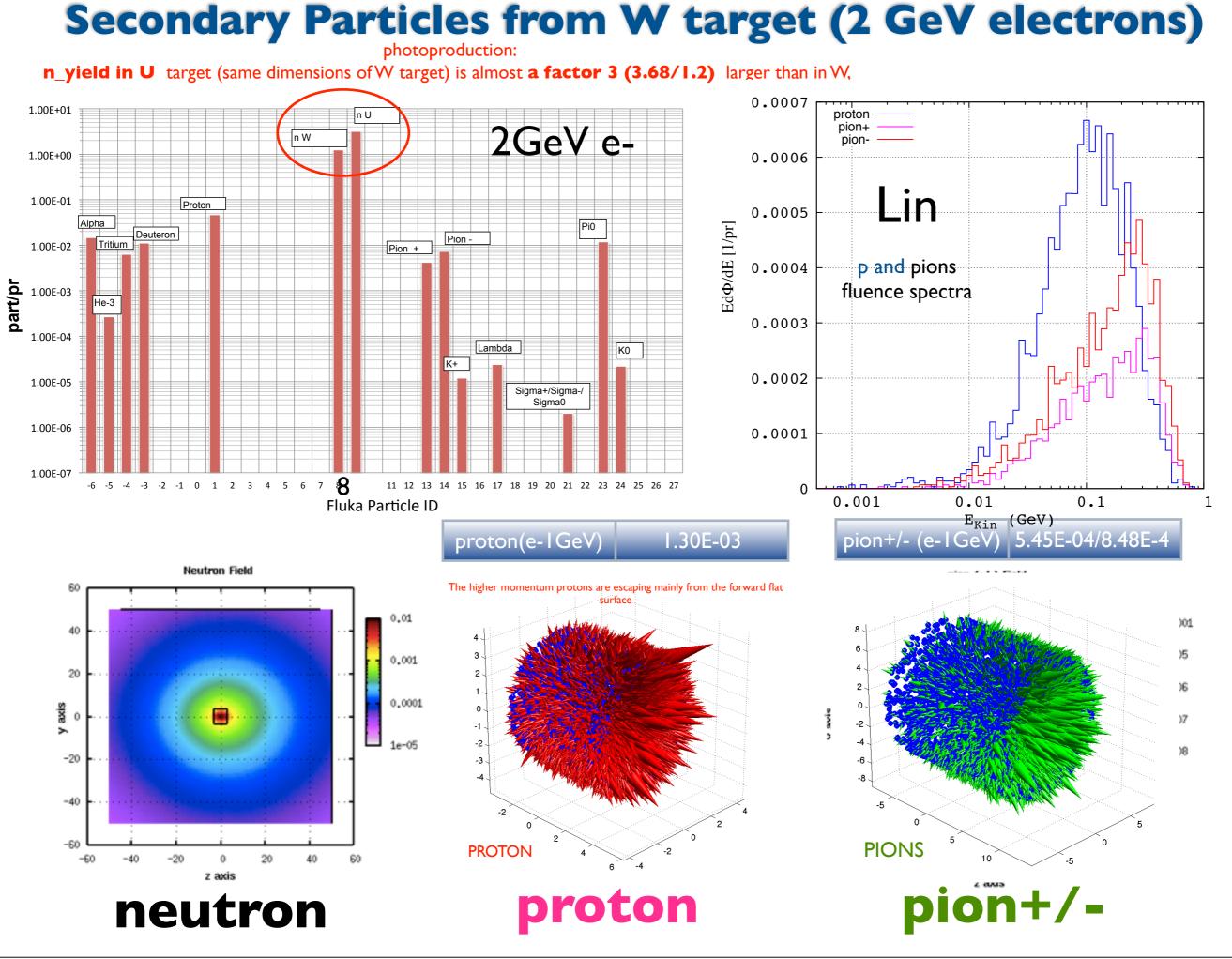
(phD student of G. Cavoto, Univ Sapienza, Rome): studying the possibility to optimize the target to produce muons (Geant4)

Carlo Mancini Terracciano

(phD student of R.Faccini. Univ Sapienza, Rome): studying the possibility to optimize the target to produce protons FLUKA







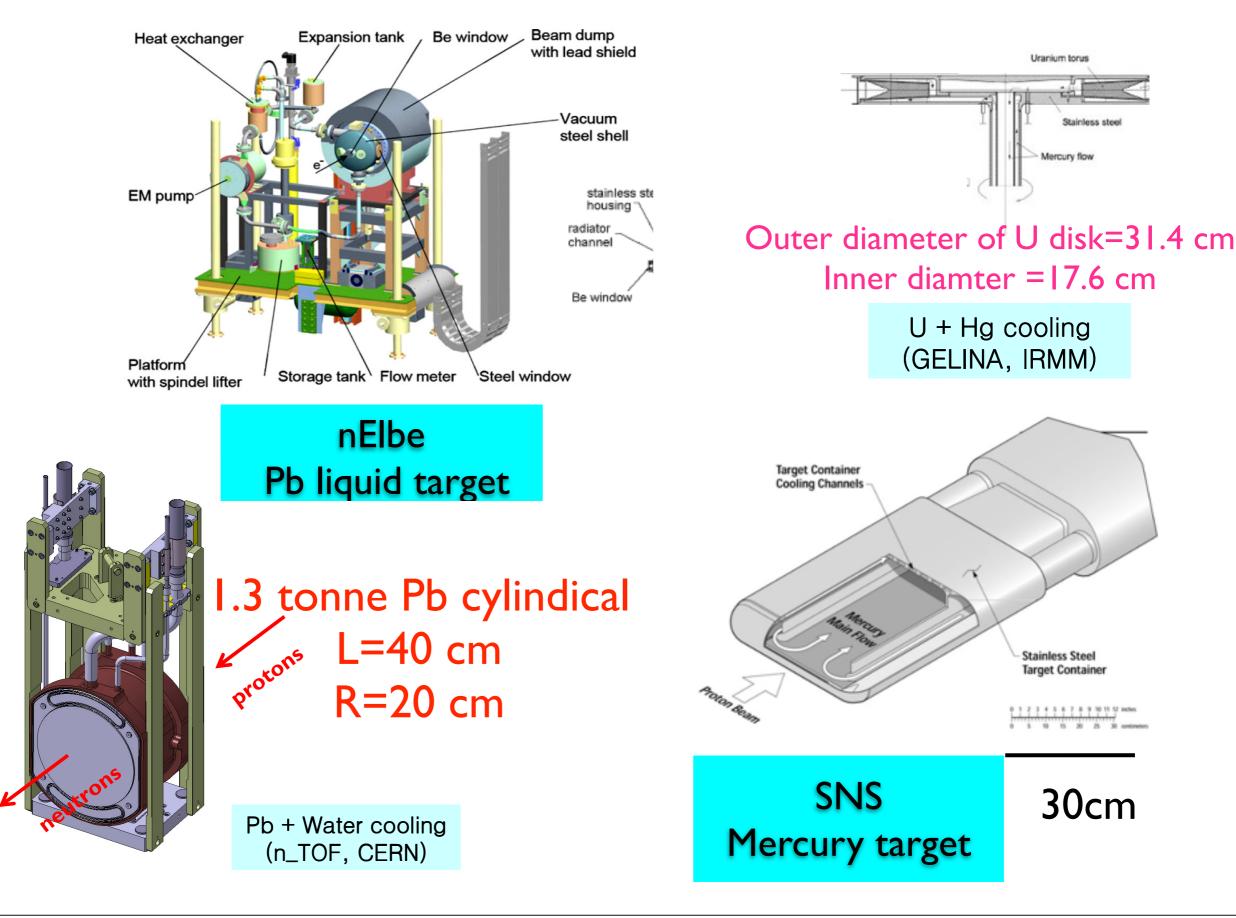


THE THERMAL PROBLEM FOR SOLID ELECTRON DUMP

Mechanical Design Service (servizio di Progettazione meccanica)

(Univ. Sapienza, Rome): thermo-mechanical design of the the target: support/cooling

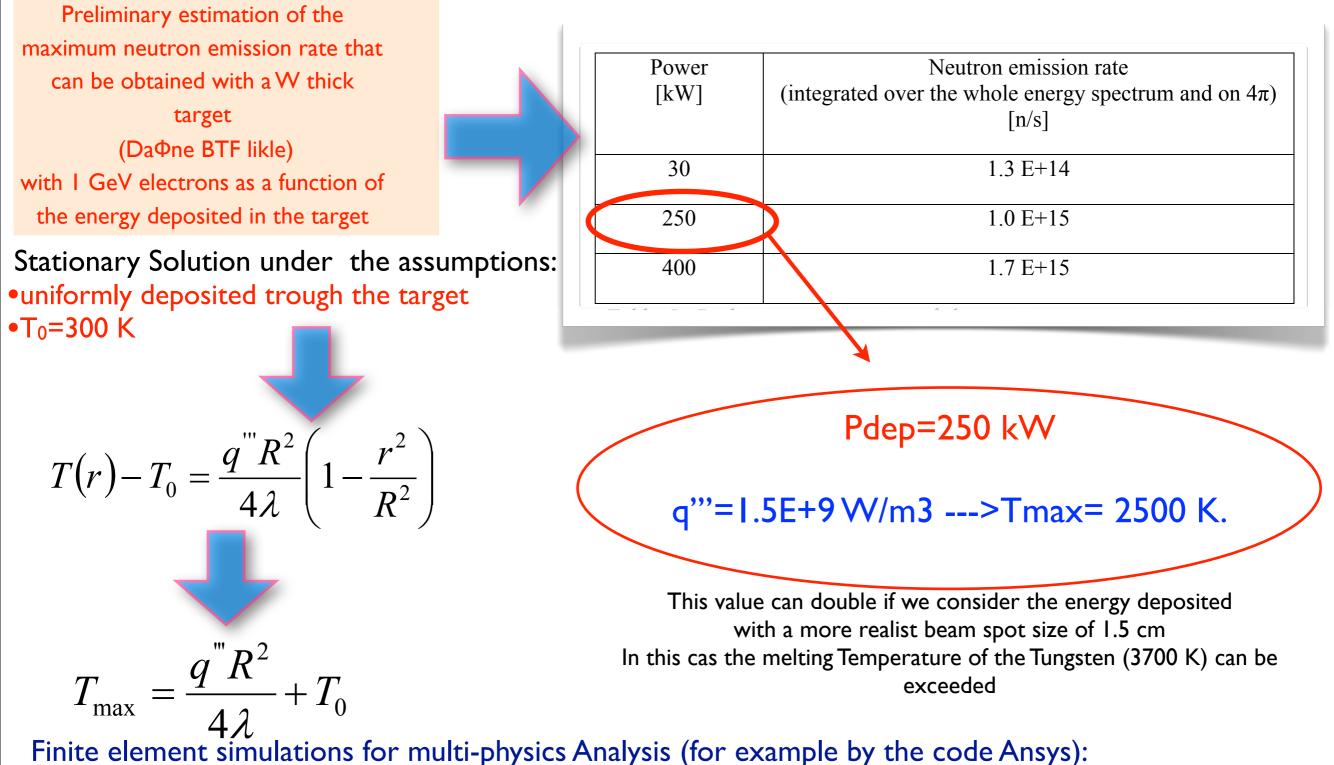
Targets of Accelerator driven Neutron sources



Target optimisation criteria

- Maximize the neutron yield with respect to the electron beam characteristics: choose the best material and geometry (es. Uranium target !!!!)
- Minimize the photon/neutron ratio
- Minimize the activation of the target and cooling system
- Keep the maximum temperature below a safe limit with respect to the melting (and other mechanical problems such as corrosion ..)
- Use a cooling system solution that preserves the emission rate and does not affect the neutron energy spectrum
- Minimize the mechanics and apparata all around the target (suitable support, displacement controller, etc..)

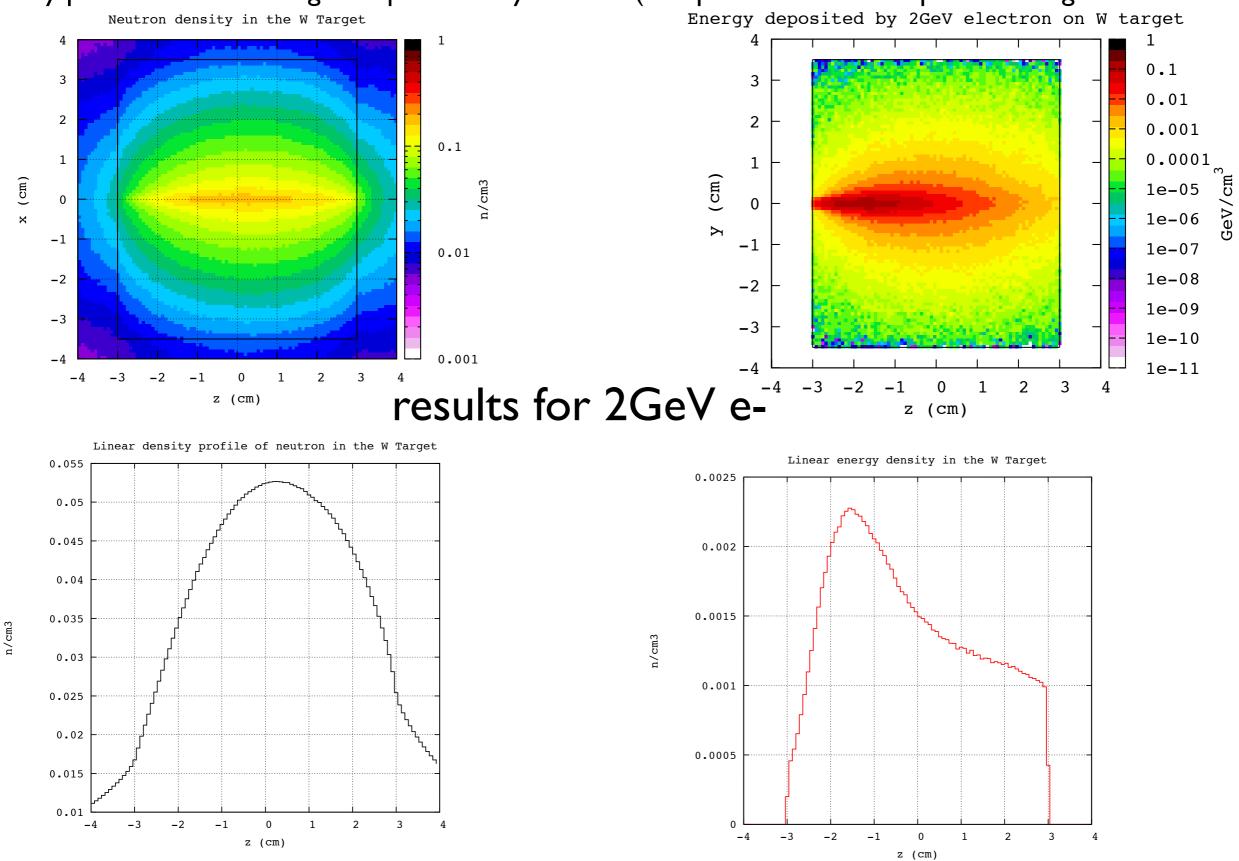
What about the need of cooling the IRIDE target: preliminar analytical estimation on W



in order to **take into account** the **most severe duty cycle** by which the target is supposed to be irradiated (time shape of the electron beam) and **the real profile of the energy deposited in the target**

W target : energy and neutron density profile

Even if the energy deposition profile has a peak in the first 2 cm from the inlet boundary the neutron density profile inside the target is quite well symmetric (as a proof of the well optimised target dimensions)



The thermal problem: material properties

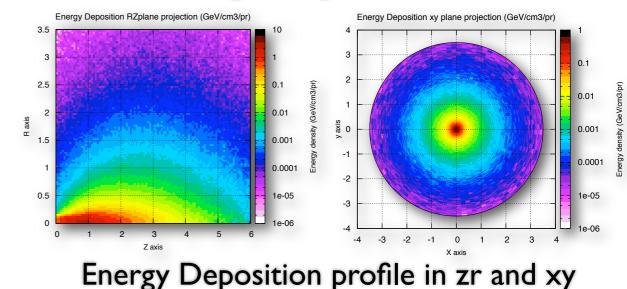
Properties (T= 300K)	Ta	W	Pb	Unat
density(g/cm3)	16.69	19.25	10.66	19.1
Z	73	74	82	92
P.M (g/mol)	180.95	183.84	207.2	238.03
Rad Length [cm]	0.41	0.25	0.56	0.32
K (thermal-cond) [W/m K]	57.5	173	35.3	27.5
E(young) [GPa]	186	411	16	208
Poisson Ratio	0.34	0.28	0.44	0.23
alpha µm/m K	6.3	15	28.9	13.9
T(melting point) [k]	3290	3695	606	1405

Table 4: Thermal and Nuclear properties of several materials

Because a suitable segmentation of the target has to be considered as a possible thermal solution, W alloys (Densimet and Inermet: 90% W+Ni+Fe) could be taken into account as valuable materials ^k (W/m-K) for the target (since much better machinable than nat-W)

Werkstoff Material	Abkürzung Abbrevation	Chemische Zusammensetzung Chemical composition [%]		Nominelle Dichte Nominal density	AMS-T-21014 Class
		W	Rest		
Schwach ferromagnetis	ch / Weakly ferromagnetic				
DENSIMET 170	D170	90	Ni, Fe	17.0	1
DENSIMET 176 / W	D176 / DW	92.5	Ni, Fe	17.6	2
DENSIMET 180	D180	95	Ni, Fe	18.0	3
DENSIMET 185	D185	97	Ni, Fe	18.5	4
DENSIMET D2M	D2M	90	Ni, Mo, Fe	17.2	-
Paramagnetisch / Para	magnetic				
INERMET 170	IT170	90	Ni, Cu	17.0	1
INERMET 176	IT176	92.5	Ni, Cu	17.6	2
INERMET 180	IT180	95	Ni, Cu	18.0	3

Typische Chemische Zusammensetzung von DENSIMET-INERMET-Standardlegierungen Typical chemical composition of DENSIMET-INERMET standard alloys



Thermal Diffusivity: index of effectiveness in heat exchange $k/(\rho C)$ in W = 3 times larger than in Ta Unfortunately it decreases increasing the bulk temperature

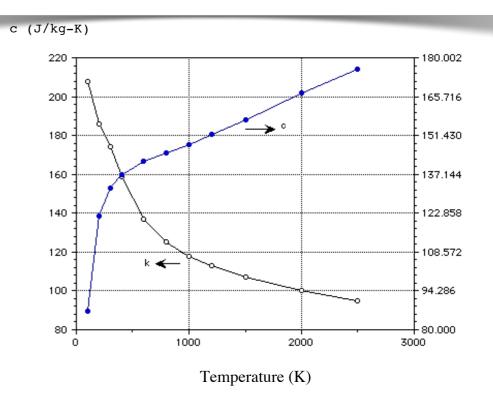


Figure 1 : Thermal conductivity and specific heat of tungsten.

Conclusion, Plans and ongoing activities

- MC codes have been validated to suitable design the optimised target
- Some quantitative and qualitatives comparisons with the major neutron sources facilities have been performed and others are in progress. Thanks to
 - Ian Anderson & Philips Ferguson from SNS
 - Jan Heyse from Gelina
 - Anna Ferrari from nElbe
 - Stuart Ansell from ISIS
- Optimal technical solutions for the target have to be envisaged in the frame of a coupled analysis neutronic-thermomechanical: in progress
- Study of the extractions line according to the main scientific and technological requirements: MC time dependent simulations (collaboration with A. Fassò and A. Ferrari) and MCStas



ENEA-INMRI



THANK YOU FOR YOUR AT ENTION