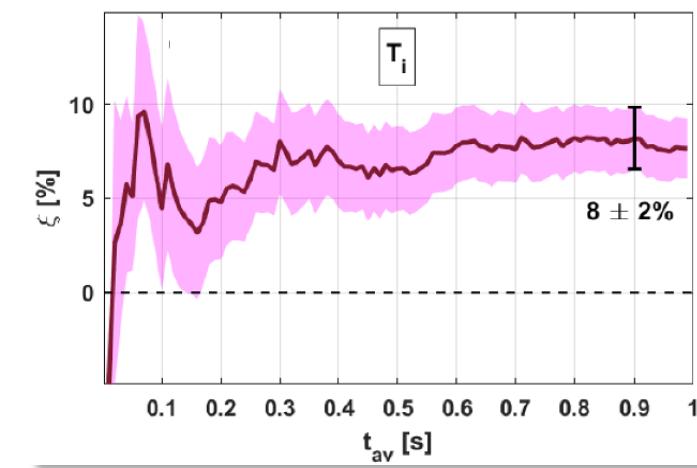
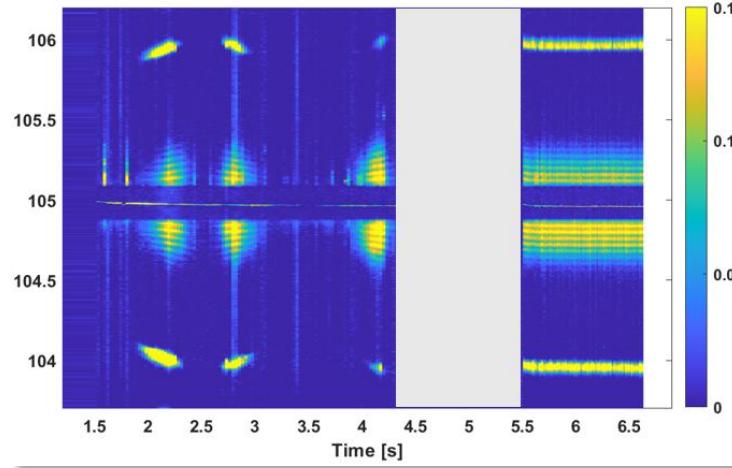
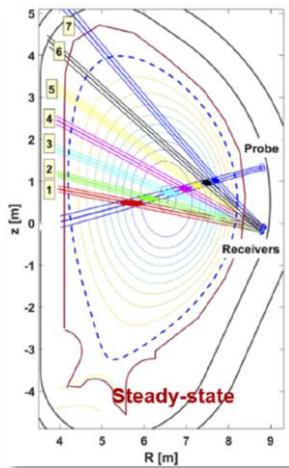


The potential of collective Thomson scattering measurements in burning-plasma devices

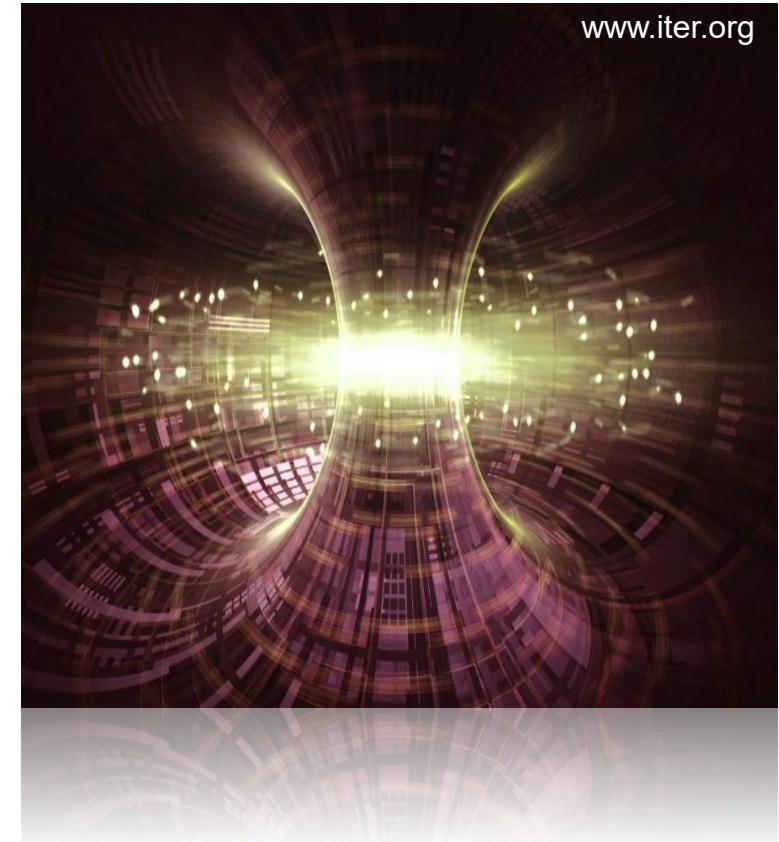
Jesper Rasmussen, Technical Univ. of Denmark

& S. B. Korsholm, J. L. Flocken, M. Jessen, M. Mentz-Jørgensen, S. K. Nielsen, R. Ragona, M. Salewski



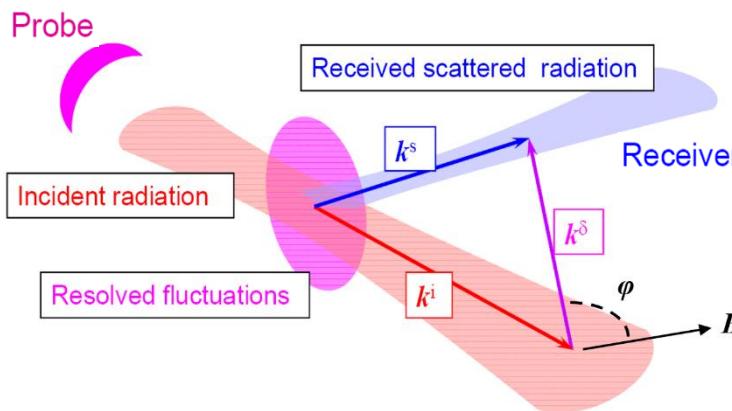
Outline

- **CTS in burning plasmas:** Why and how?
- **Possible measurements** in ITER, DEMO, SPARC, STEP (+ JT60-SA)
- **Challenges and opportunities** with CTS in the burning-plasma era
- **Summary**



Collective Thomson scattering in burning plasmas: How and why?

Principle of CTS



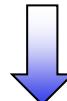
Need:

$$\alpha_s = \frac{1}{|\mathbf{k}^\delta| \lambda_D} > 1 \rightarrow \text{microwaves}$$

High P_{probe} → gyrotrons

Possible measurements

$$g(u) = \int f(\mathbf{v}) \delta\left(\frac{\mathbf{v} \cdot \mathbf{k}^\delta}{k^\delta} - u\right) d\mathbf{v}$$

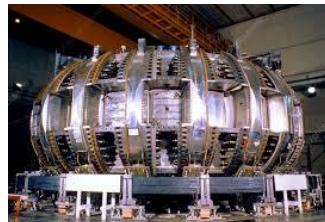
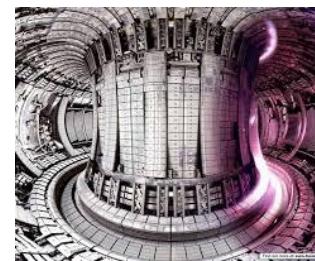
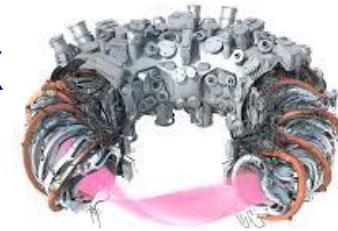
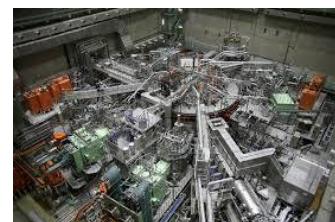


- T_i, v_{tor}
- $f_{\text{fast}}(u = v_{1D})$
- For $\varphi \approx 90^\circ$:
 $n_D/n_T, n_{\text{He-3}}, n_{\text{imp}}$

for confined ions – spatially resolved.

Reactor relevance

- Versatility: Non-invasive monitoring of
 - fusion fuel
 - fusion products
 - kinetic profiles in plasma core
- No optical components, no enhanced background
[Rasmussen+ 2022 EPS O4.112]
- No reliance on NBI

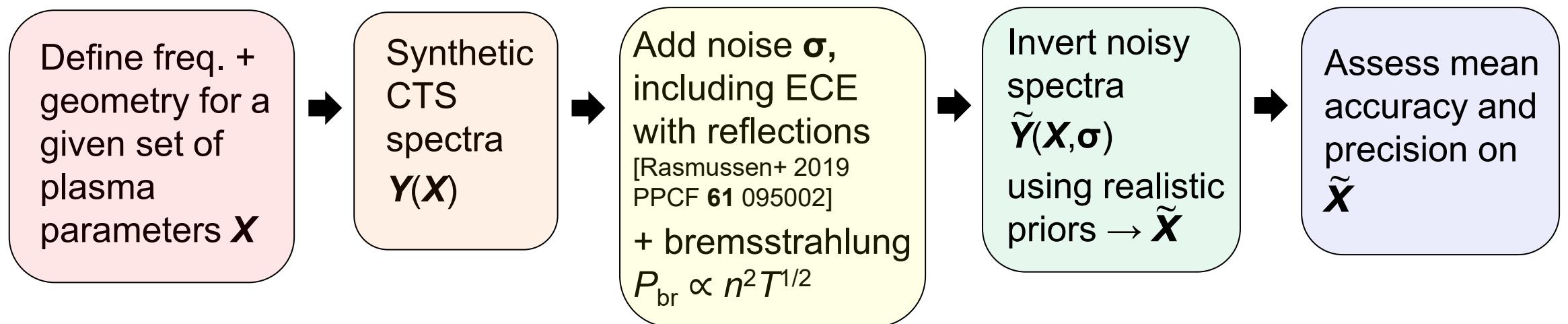
ASDEX-U[Meo+ 2008 *RSI* **79** 10E501]**TEXTOR**[Bindslev+ 2006 *PRL* **97** 205005]**TFTR**[Machuzak+ 1995 *RSI* **66** 484]**JET**[Bindslev+ 1999 *PRL* **83** 3206]**FTU**[Tartari+ 2007 *RSI* **78** 043506]**W7-AS**[Suvorov+ 1995 *PPCF* **37** 1207]**W7-X**[Moseev+ 2019 *RSI* **90** 013503]**LHD**[Kubo+ 2010 *RSI* **81** 10D535]And soon: **TCV** [Meier+ in prep.]

Two main considerations for CTS diagnostic design optimization

Minimize background noise = mainly ECE → operate between/below ECE harmonics

Minimize refraction = sensitivity to plasma conditions → operate away from cutoffs

Procedure



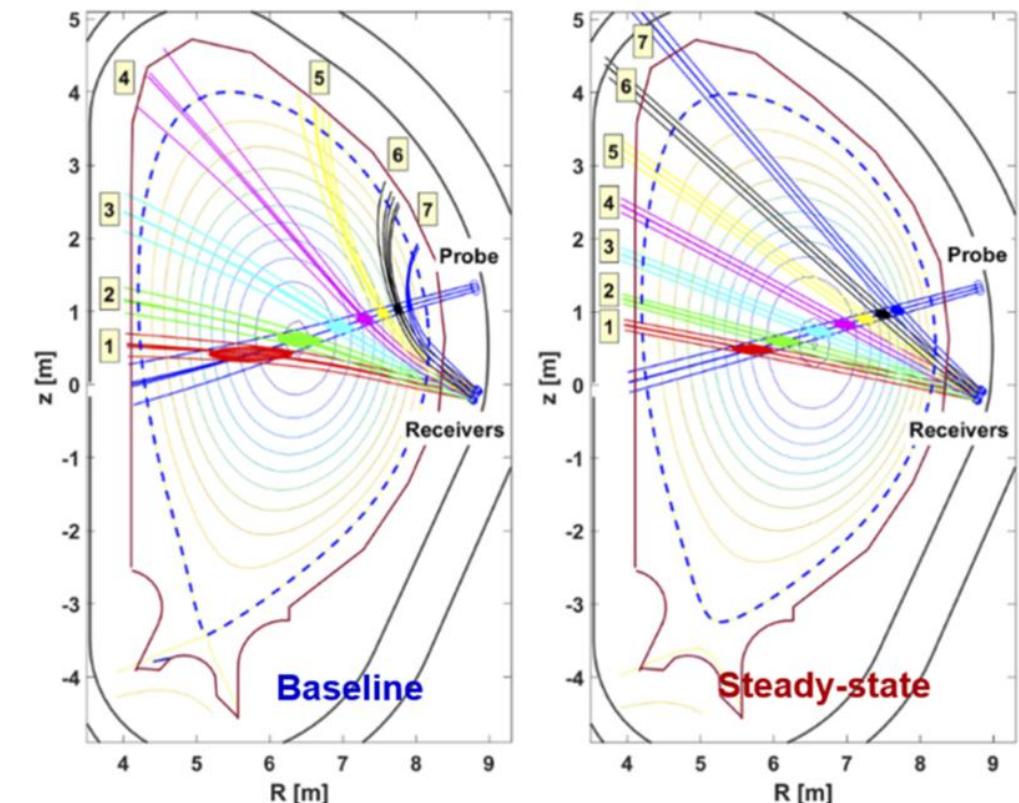
ITER CTS fully designed under contract with F4E

Goal: Contribute to measurements of density profiles and "energy" spectra of

- Fusion alphas
- Fast p, D, T, ^3He from fusion or auxiliary heating

Measurement requirements: n_α to 20% at

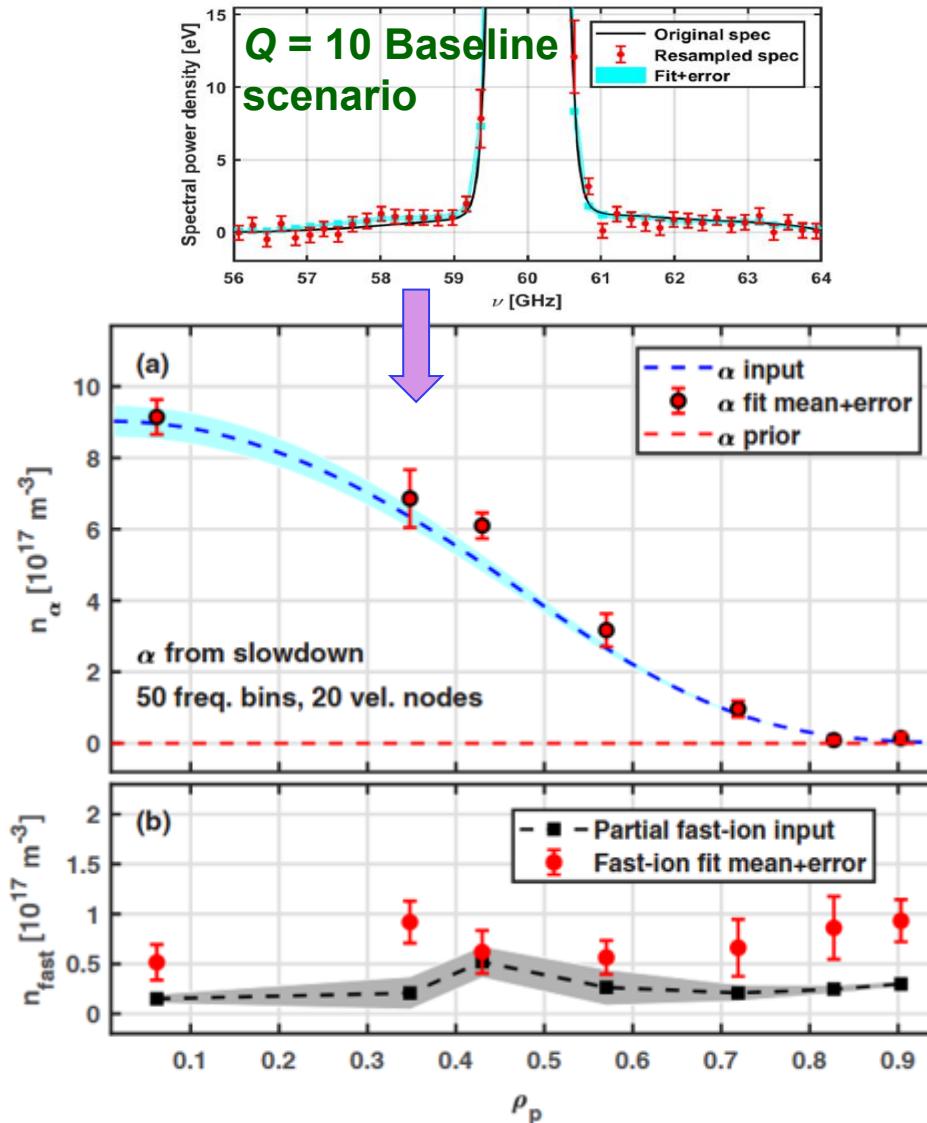
- $\Delta t = 100$ ms
- $\Delta R = 10\text{--}100$ cm



Design: Subharmonic system, 60 GHz X-mode

[Korsholm+ 2022 *RSI* **93** 103539]; see also [Orsitto & Giruzzi 1997 *RSI* **68** 686]

ITER CTS fusion alpha measurements meet the requirements



Inferred accuracy on n_α : 10–15% at $\Delta t = 100$ ms

$n_\alpha(\rho, u)$ for monitoring of

- $S_{\text{fus}} = 3E_\alpha n_\alpha [\tau_{\text{SD}} \ln(1 + (\nu_b/\nu_c)^3)]^{-1}$ [W/m³]
- α transport in real- and velocity space
→ interactions with MHD modes

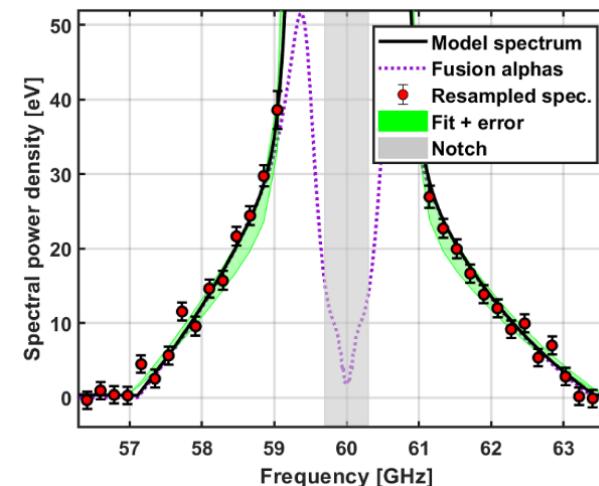
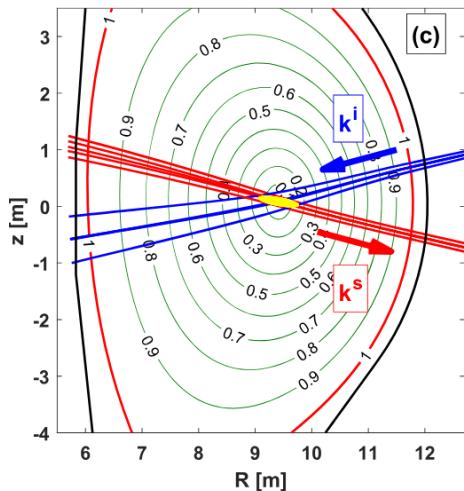
Simultaneous monitoring of other MeV ions
is feasible (e.g. $n_{D,\text{NBI}}$ to < 15%)

[Rasmussen+ 2019 NF 59 096051]

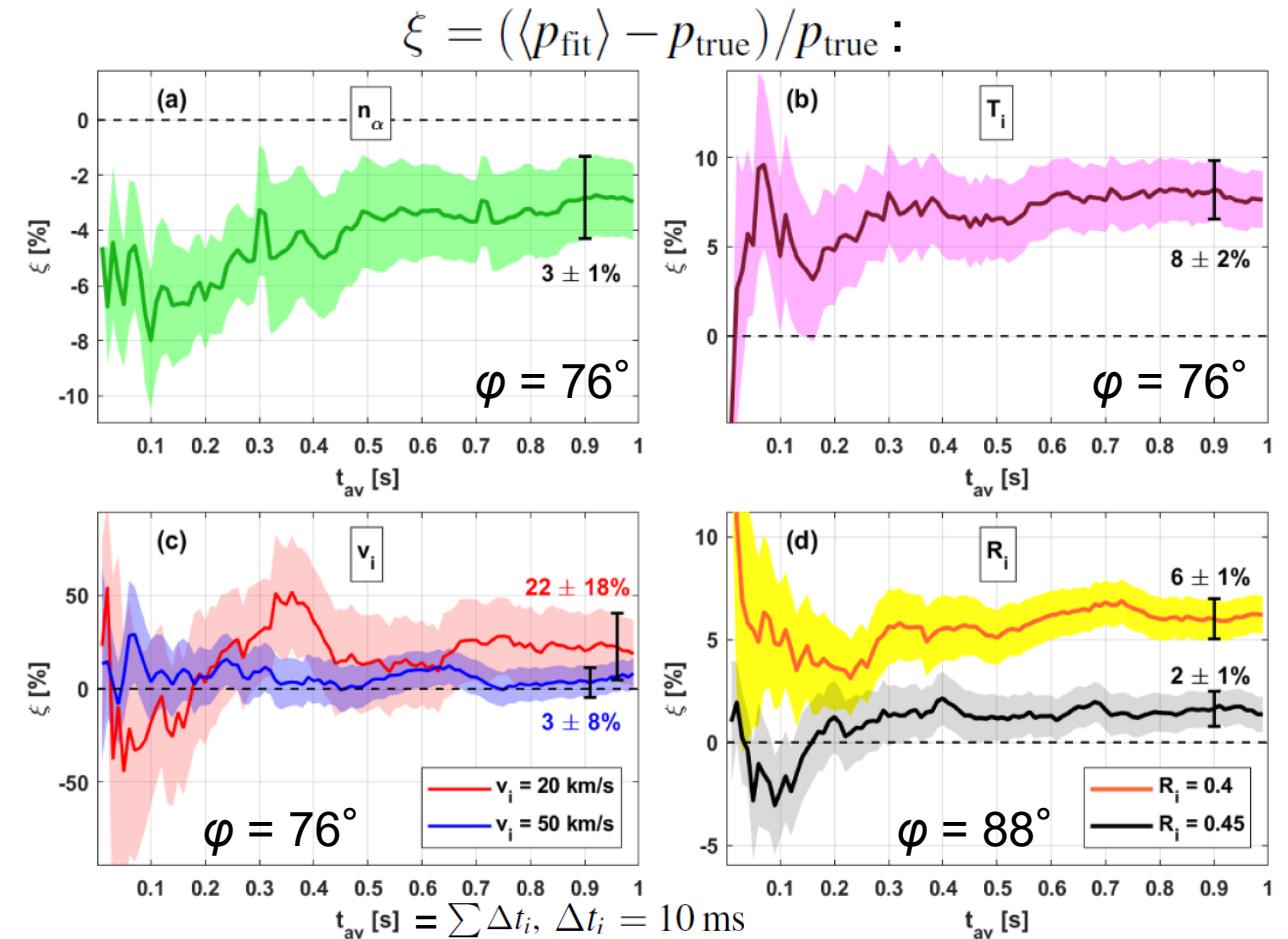
EU-DEMO “2019 G1 baseline” configuration: Core CTS also feasible

ITER-like setup:

60 GHz subharmonic X-mode



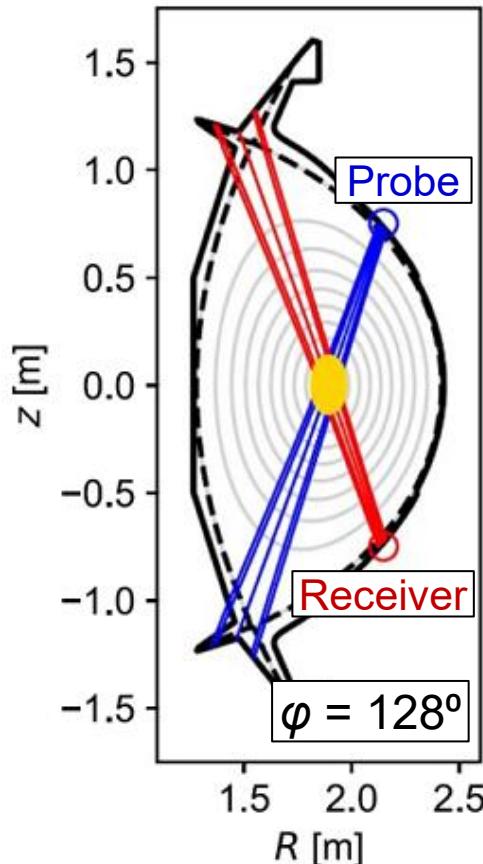
[Rasmussen & Korsholm 2024 NF 64 046003]



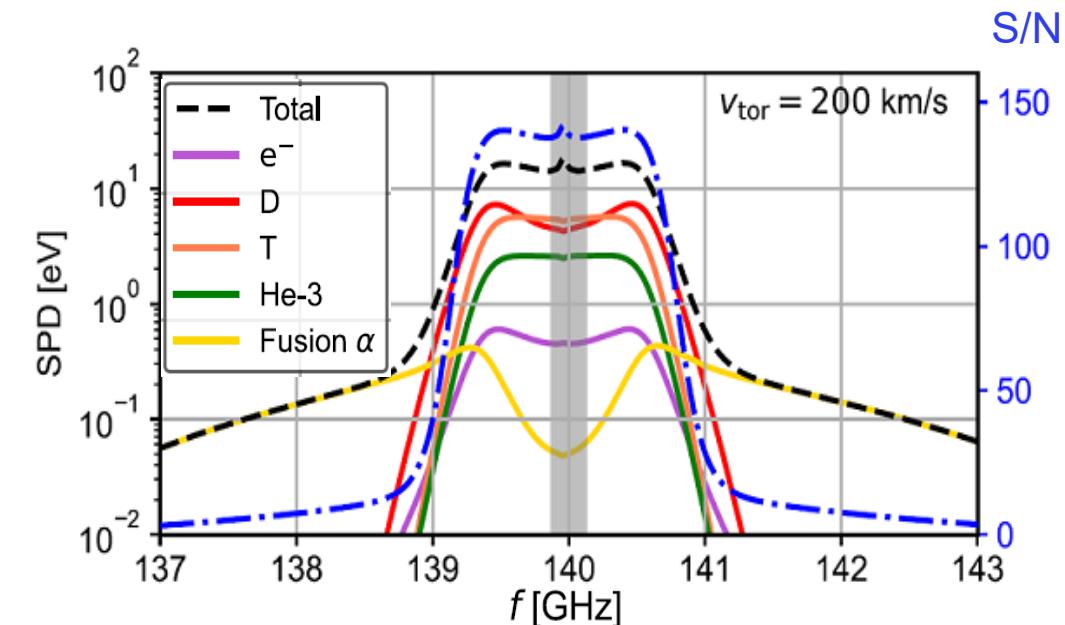
For the 2024 DEMO low aspect ratio config: See talk by J. Flocken on Friday

CTS in an ICRH-only device: SPARC as a promising case

SPARC operation 2027 → short timeline → use available gyrotron tech. → 140 GHz X-mode

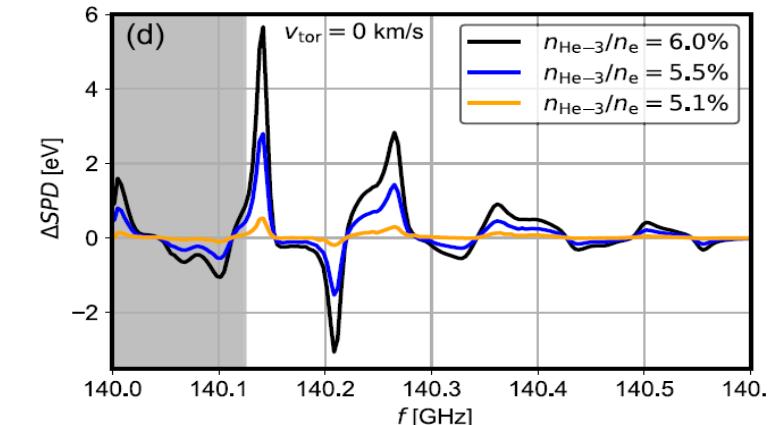
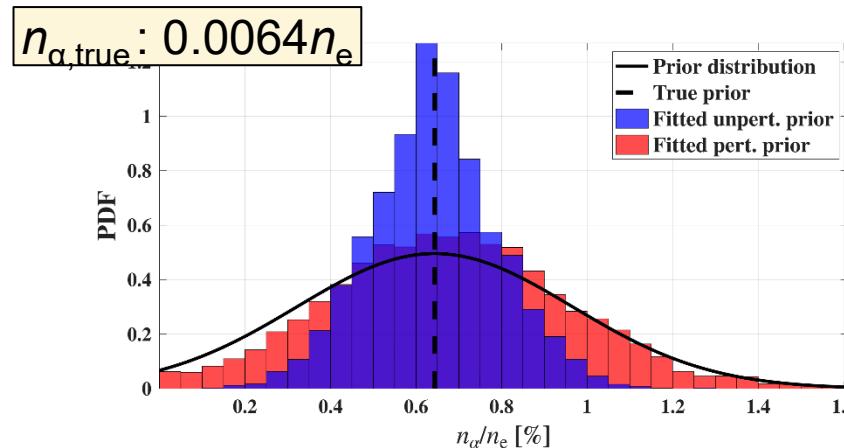
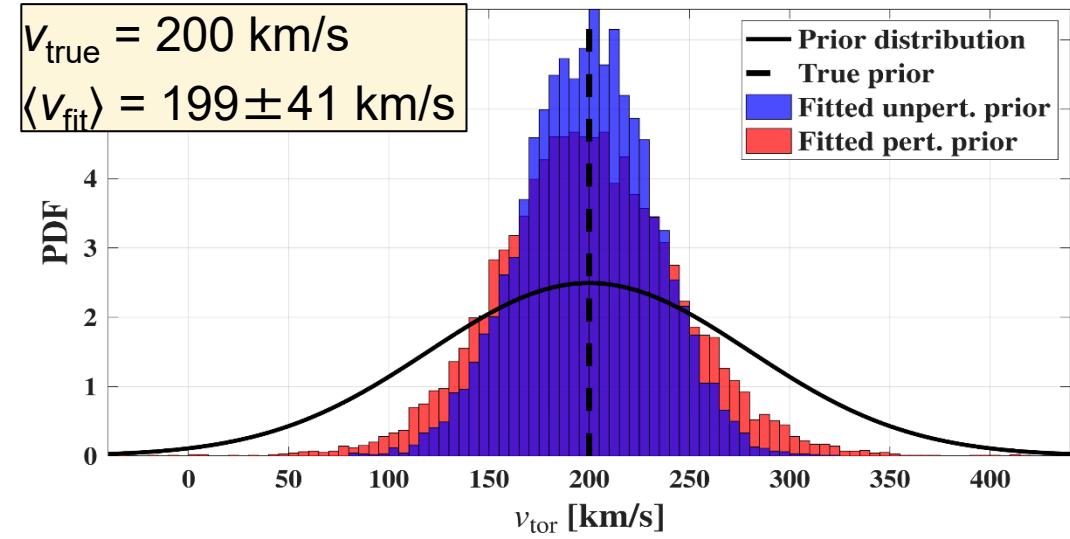
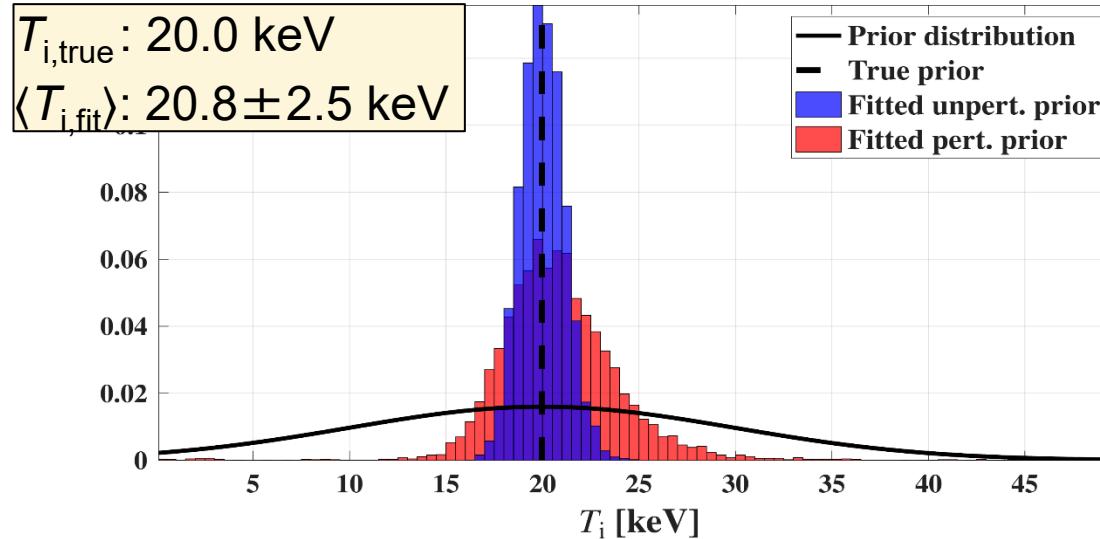


- $Q = 11$ Primary Reference Discharge
 - $B_t \approx 12.5$ T → distance to
 - X_L cutoff ≈ 60 GHz
 - $n = 1$ ECR $\gtrapprox 250$ GHz
- ↓
- No refraction, CTS background noise dominated by bremsstrahlung



[Menz-Jørgensen+ 2025 NF 65 046028]

Preliminary results: Good reconstruction of bulk-ion properties



Variations in
He-3 content
(default: 5%)

Work by M. Mentz-Jørgensen

CTS in a spherical tokamak? The (challenging) case for STEP CTS

STEP "SPP-001" configuration

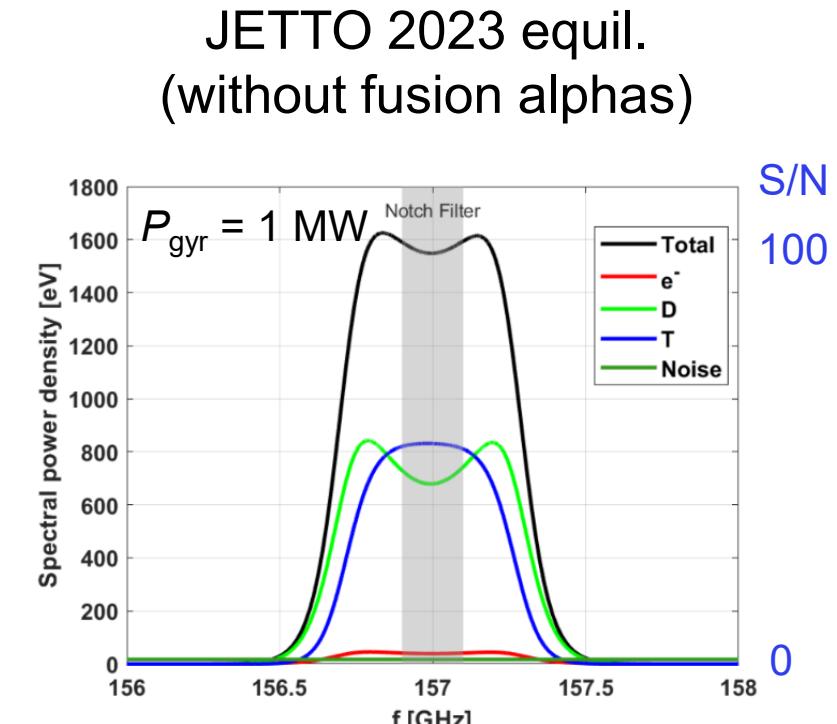
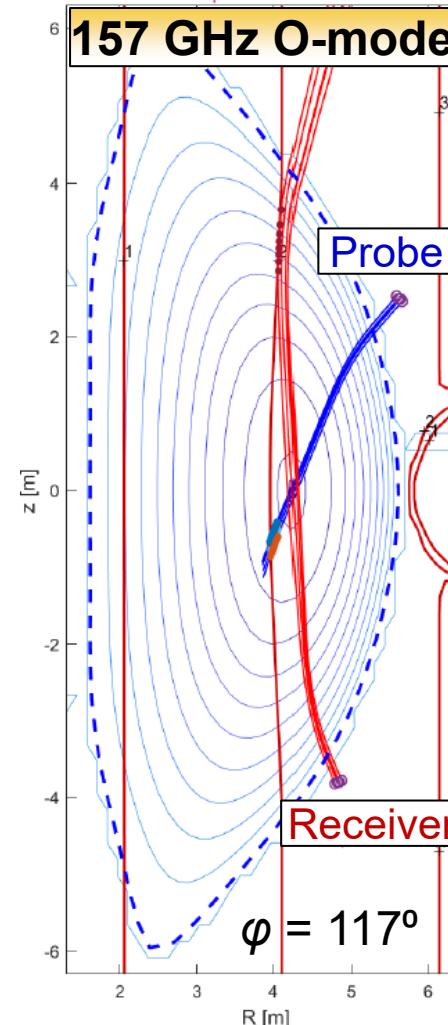
Challenges:

- $n_{e,0} \approx 2 \times 10^{20} \text{ m}^{-3}$ → high cutoff freq.
- Low $B_t = 3.2 \text{ T}$ and low $A = R/a \approx 1.8$
→ ECE harmonic overlap



Subharmonic setup not an option
+ near-vertical geometry required

Note: Port locations not yet defined

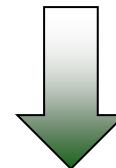


2025: New "SPP-002" config:
 $R: 3.6 \rightarrow 4.3 \text{ m}$, $n_{e,0} \rightarrow 1.3 \times 10^{20} \text{ m}^{-3}$
[Olde+ 2025 EPS 4P.275]

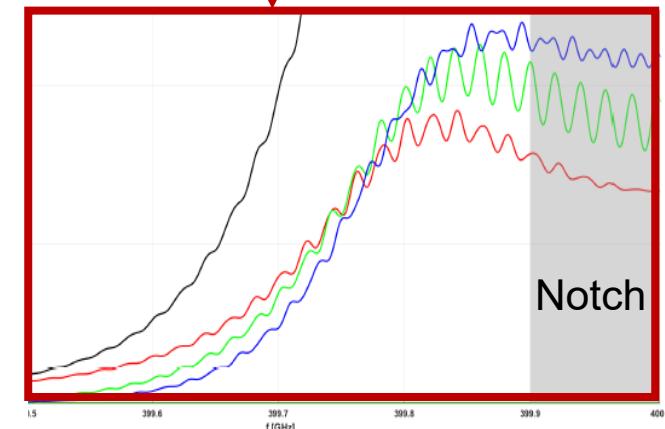
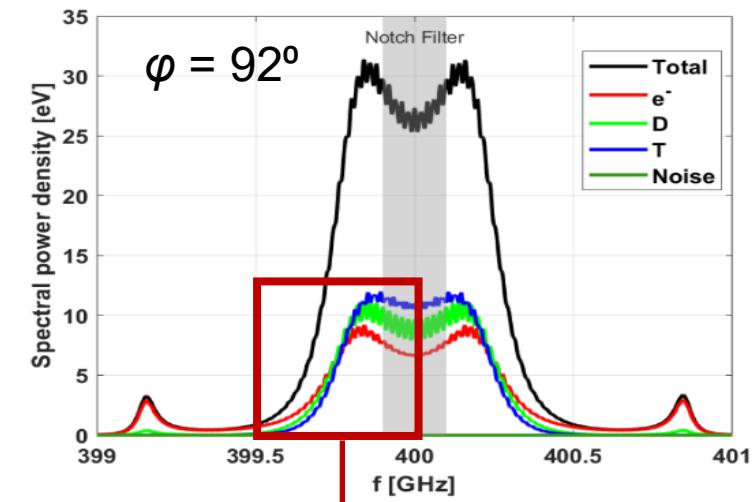
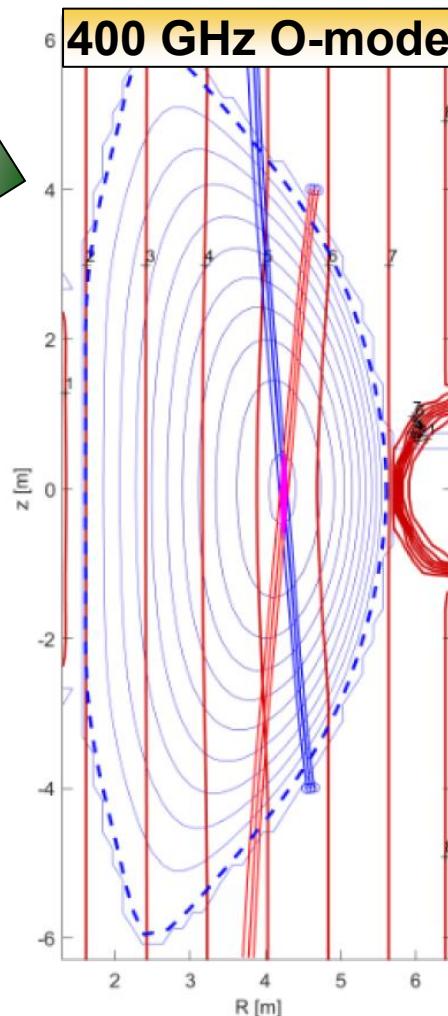
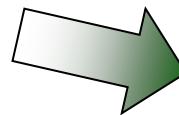
Work by V. Esposito + M. Bøtker-Rasmussen

STEP: Possible alternatives available at ~sub-THz frequencies

- 400 GHz $P \approx 100$ kW gyrotron
 [Saito+ 2010 *J. Phys.: Conf. Ser.* **227** 012013;
 2019 *Plasma Fusion Res.* **14** 1406104]
- Vertical/toroidal laser-based
 ~1 THz setup

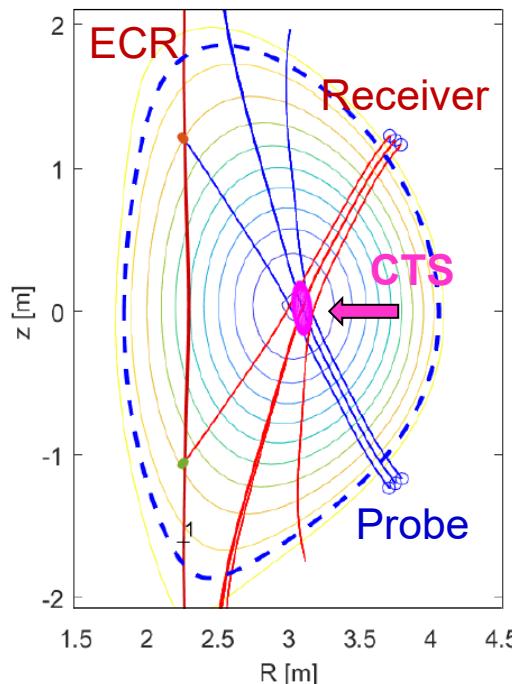


O_b [/m]	ρ	φ [$^\circ$]	α_s	T_{rad} [keV]
68	0.00	88	4.1	0.5
49	0.03	132	7.7	0.01

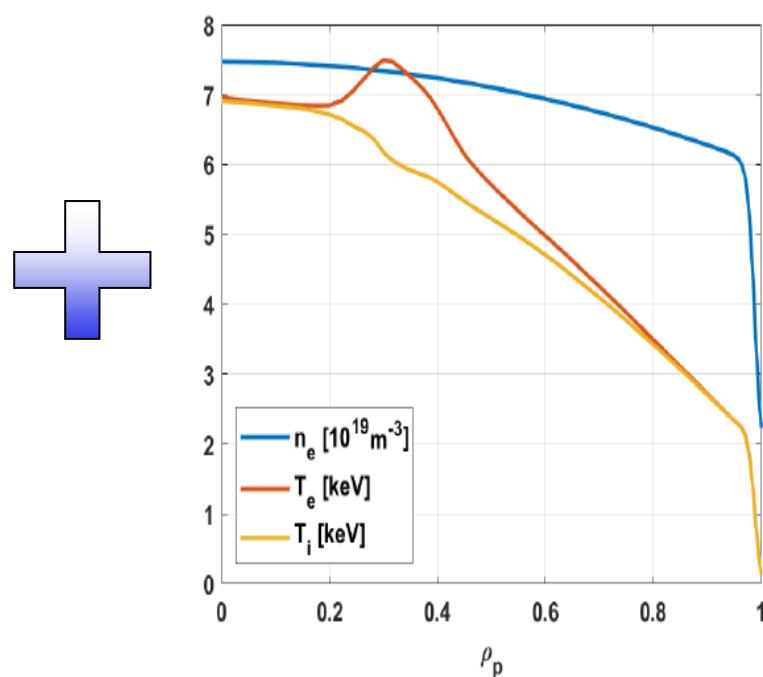


JT60-SA: Initial study confirms feasibility of fast-ion CTS

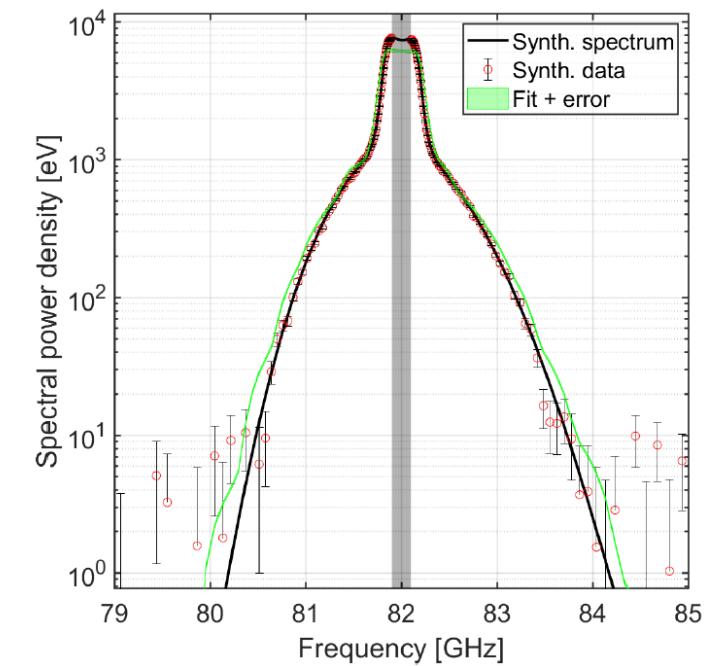
Potential implementation



EFIT BaselineScenarioOP2



$n_{\text{fast}} = n_{D,\text{NBI}}$ to $\sim 10\%$, T_i to $\sim 5\%$



82 GHz, O-mode: $\varphi = 67^\circ / 92^\circ$

Using existing ECRH infrastructure,
port P-1, P-4, P-8, or P-11

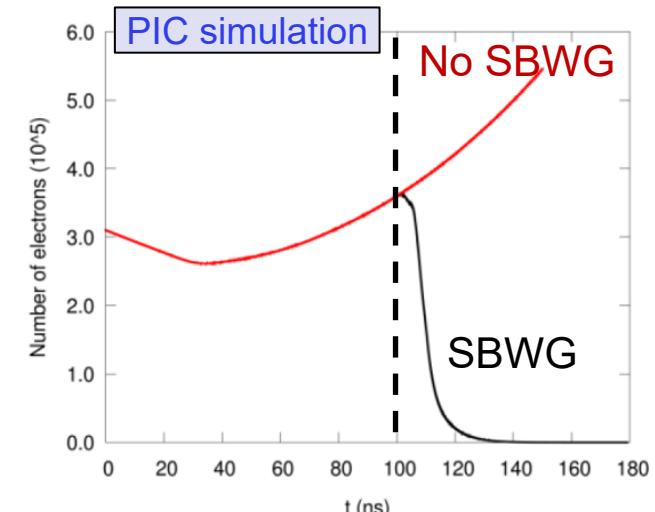
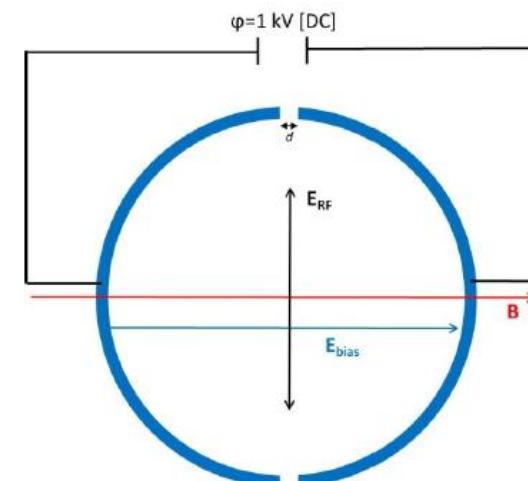
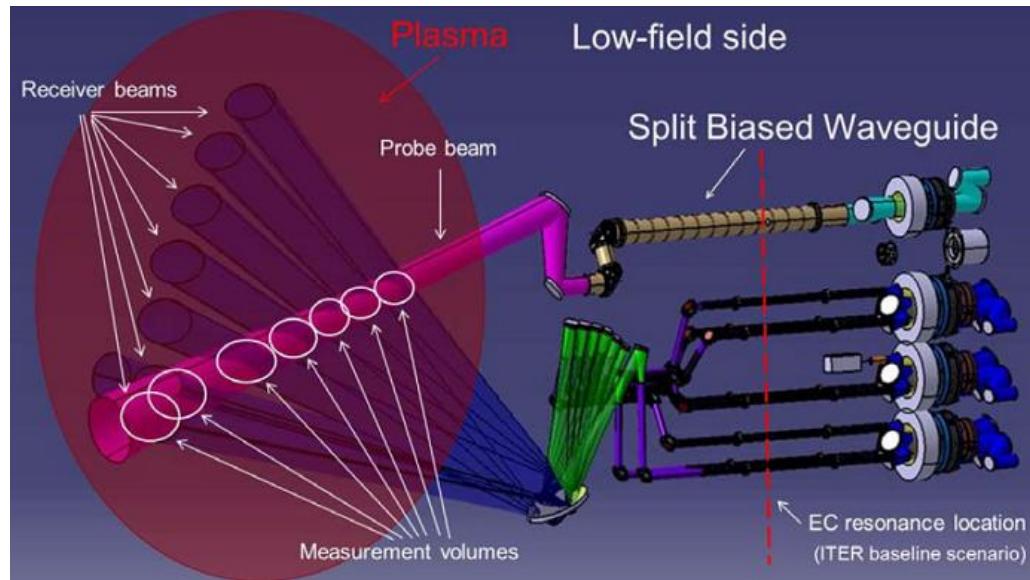
But need more plasma equilibria for sensitivity study
May require: 82 GHz \rightarrow 110 GHz due to cutoff

Work by E. A. Christensen

Operational considerations: Challenges in burning-plasma devices (1)

Risk of local breakdown:

ITER, DEMO, SPARC: Subharmonic setup, with $n = 1$ ECR location in gyrotron waveguide

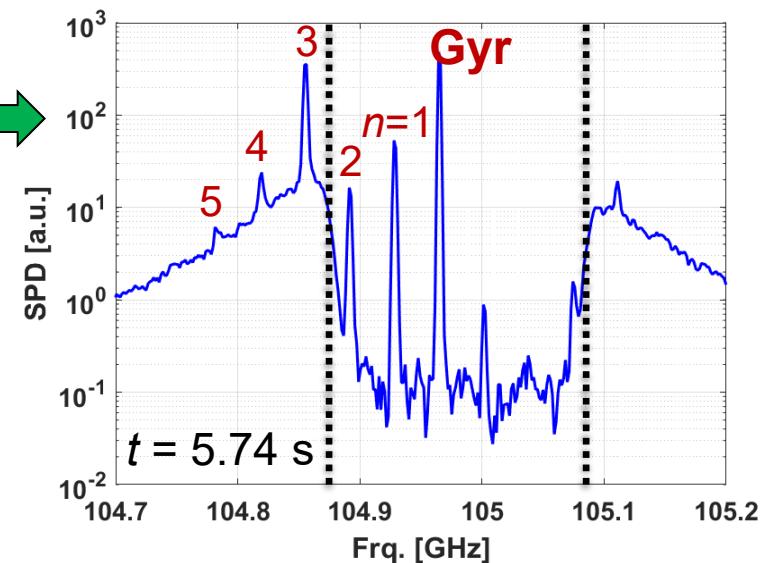
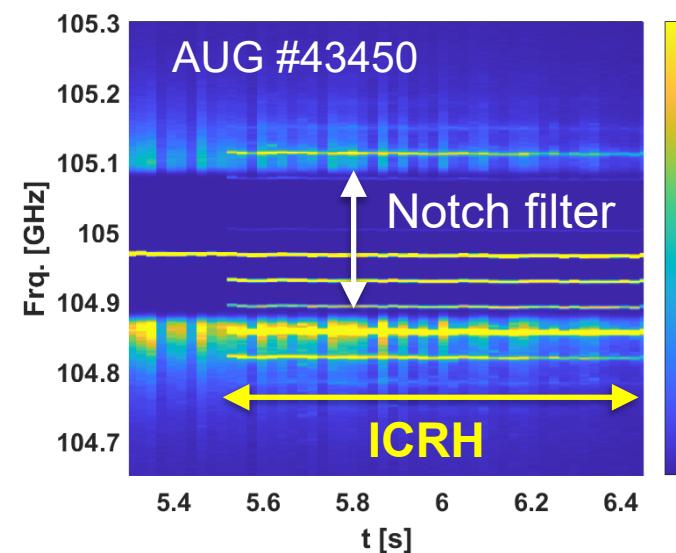
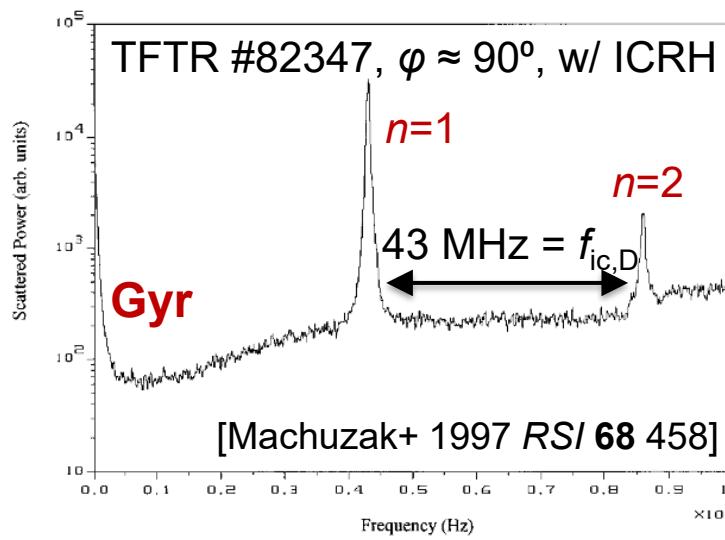


Planned mitigation at ITER: Split biased waveguide [Larsen+ 2019 JINST 14 C11009]

Operational considerations: Challenges in burning-plasma devices (2)

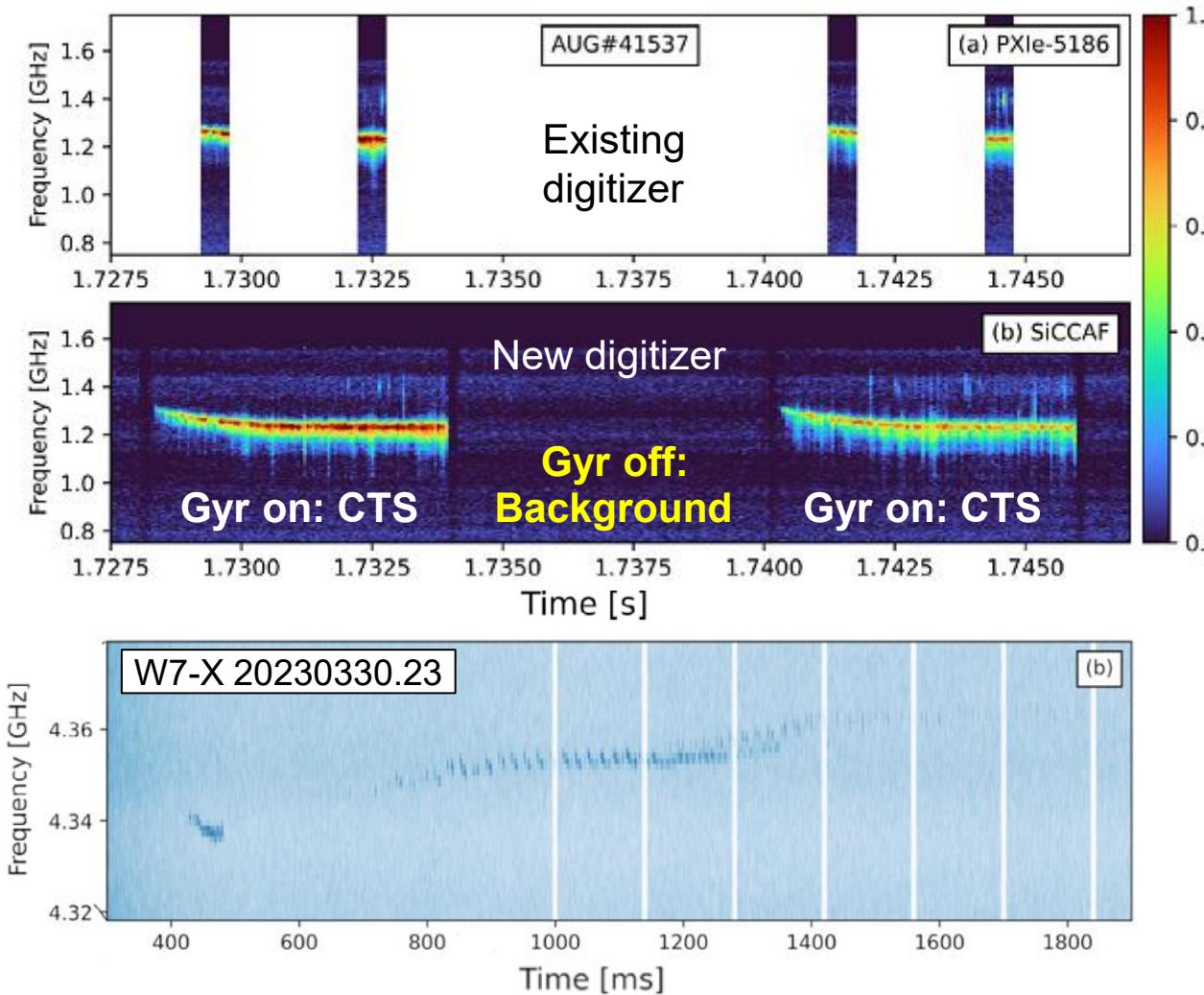
Induced emission by MeV ions (for $\varphi \approx 90^\circ$) or non-linear gyrotron–ICRH wave mixing

[TFTR: Machuzak+ 1997 *RSI* **68** 458], [TEXTOR: Woskov+ 2006 *RSI* **77** 10E524], [AUG: Meo+ 2010 *J. Phys. Conf. Ser.* **227** 01201]



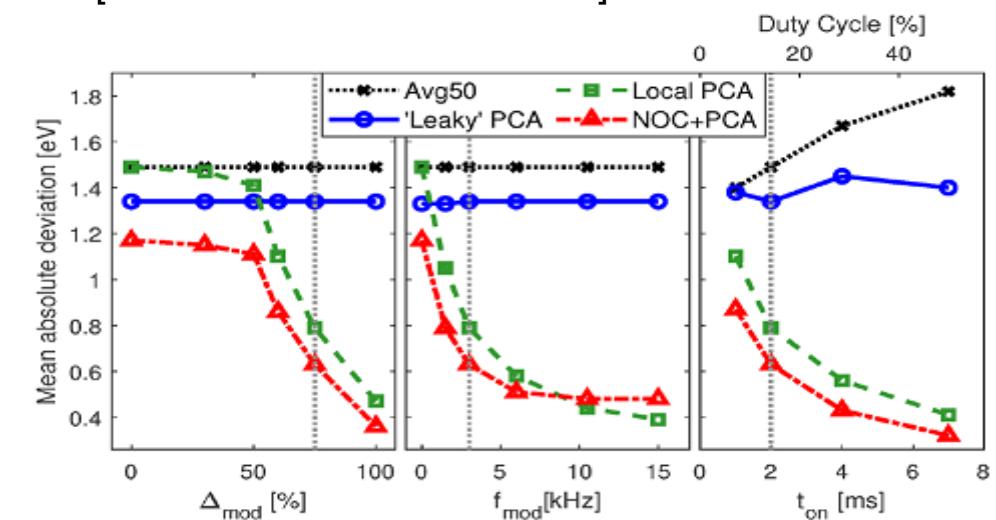
Possible mitigation: **Passive CTS receiver** [Nielsen+ 2015 *PPCF* **57** 035009], **probe gyrotron power modulation** [Verdier+ 2025 *RSI* **96** 043513], **tunable notch filters** [Acar+ 2016 *Int. J. Microw. Wirel. Tech.* **8** 567]

Promising prospects for near-continuous CTS measurements



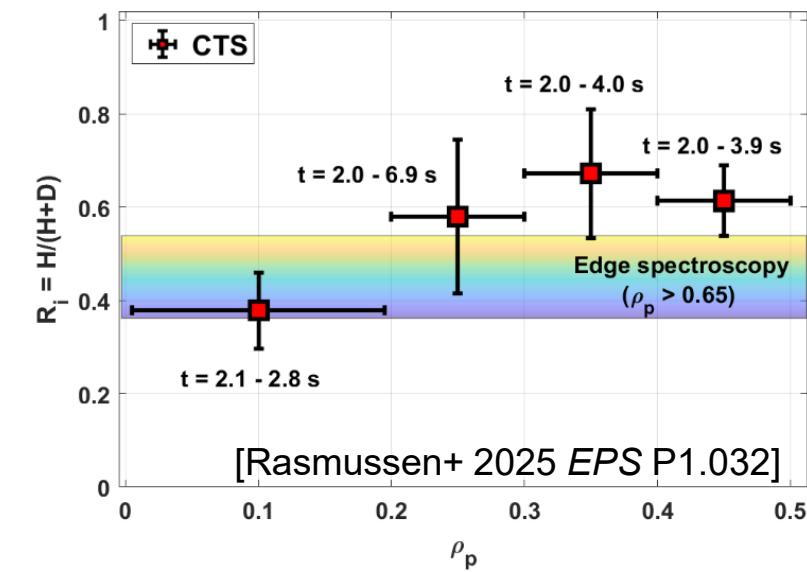
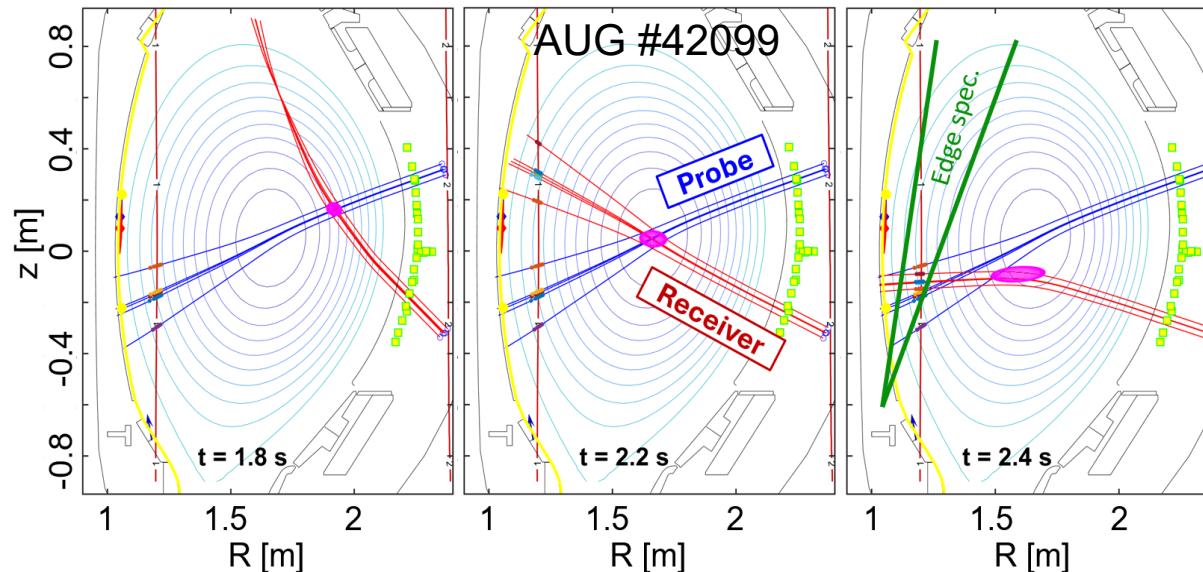
FPGA-based continuous fast digitizers now becoming available
[Verdier+ 2024 FED 206 114597]

Novel CTS background techniques also allow longer acquisition pulses
[Verdier+ 2025 RSI 96 043513]



Opportunities for CTS-based real-time control of burning plasmas

ASDEX-U CTS: First core measurements of n_H/n_D in a purely ECRH-heated plasma:



- Preliminary results based on a multilayer perceptron neural network
- Prospects towards rapid inference using continuous FPGA-based digitizers → **real-time control** [Verdier+ 2024 FED 206 114597]

See also [Van den Berg+ 2018 RS/ 89 083507; Stylianidis 2019 MSc Thesis TU/e; Nishiura+ 2020 JINST 15 C01002; Zhang+ 2024 RS/ 95 093542]

Overview: CTS can monitor key physics/operational parameters

Across burning-plasma devices, CTS can contribute to monitoring

- P_{fus} and alpha transport through measurements of $n_{\alpha}(\rho, v_{1D})$
- Fast D, T, H, ... distribution functions (from fusion or auxiliary heating)
- Core ion heat transport (T_i) and toroidal rotation (instabilities, H-mode access)
- Fuel-ion ratios → optimization of P_{fus} + pellet fuelling scheme (burn control)
- He minority concentrations (He ash, optimization of auxiliary heating)
- ELMs (for mitigation / pellet pacing) [Verdier+ 2025 RSI **96** 043513]

on timescales $t < 0.1\tau_E \sim 0.1\tau_{\text{slowdown},\alpha}$ (ITER/DEMO)

Summary and outlook

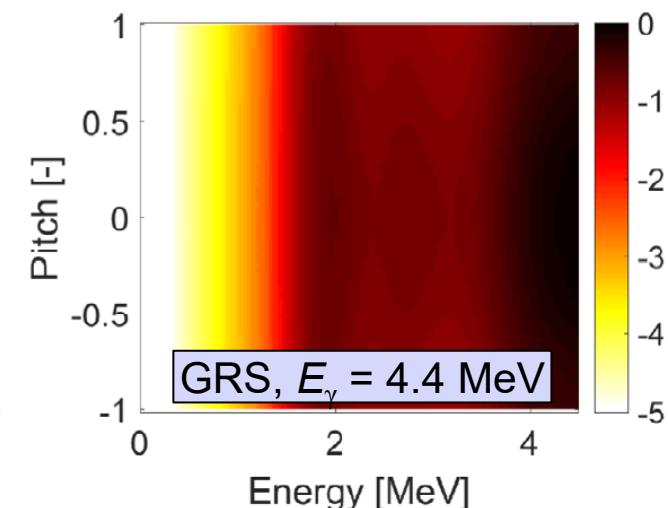
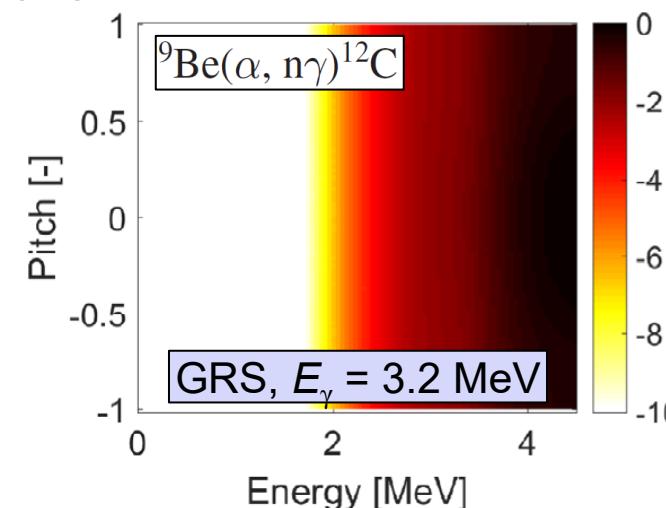
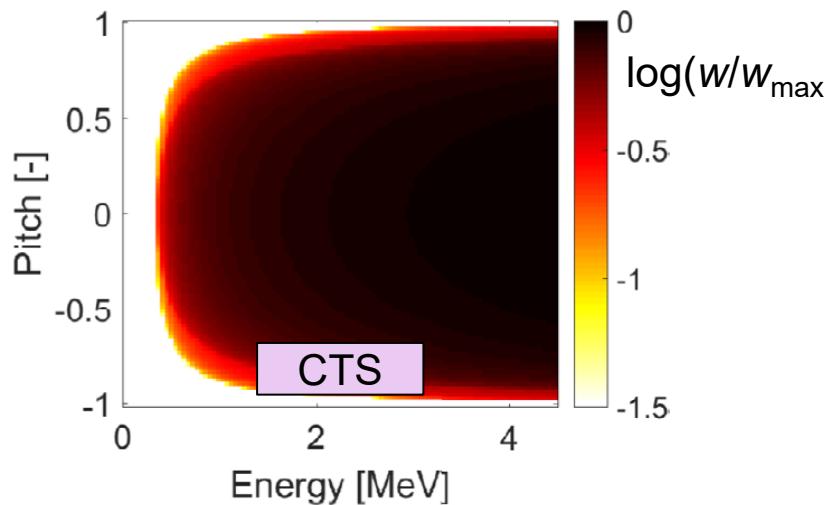
- CTS can play pivotal role in monitoring key plasma parameters in burning plasmas → optimization of machine performance and safety
- Diagnostic being built for ITER, useful setups also feasible at DEMO, SPARC, JT60-SA – and perhaps STEP
- Ongoing soft- and hardware developments → promising prospects for using CTS for continuous real-time control of burning plasmas

Backup

The fusion alpha energy distribution is not directly accessible by CTS

No single-view diagnostic can directly measure E of confined fusion alphas

Weight functions w : $S = \int \int w f dv_{\parallel} dv_{\perp}$ = diagnostic **observation regions**:



- CTS the only diagnostic to measure ITER fusion alphas at **all** E (also with W wall)
- α energy spectrum inferable if assuming e.g. *isotropy* in velocity space

[Salewski+ 2018 *NF* **58** 096019; 2025 *NF* submitted], [Ciurlino+ 2025 *EPS* 2P.126]