



Bayesian Uncertainty Quantification for Radiation Transport Calculations at FRIB

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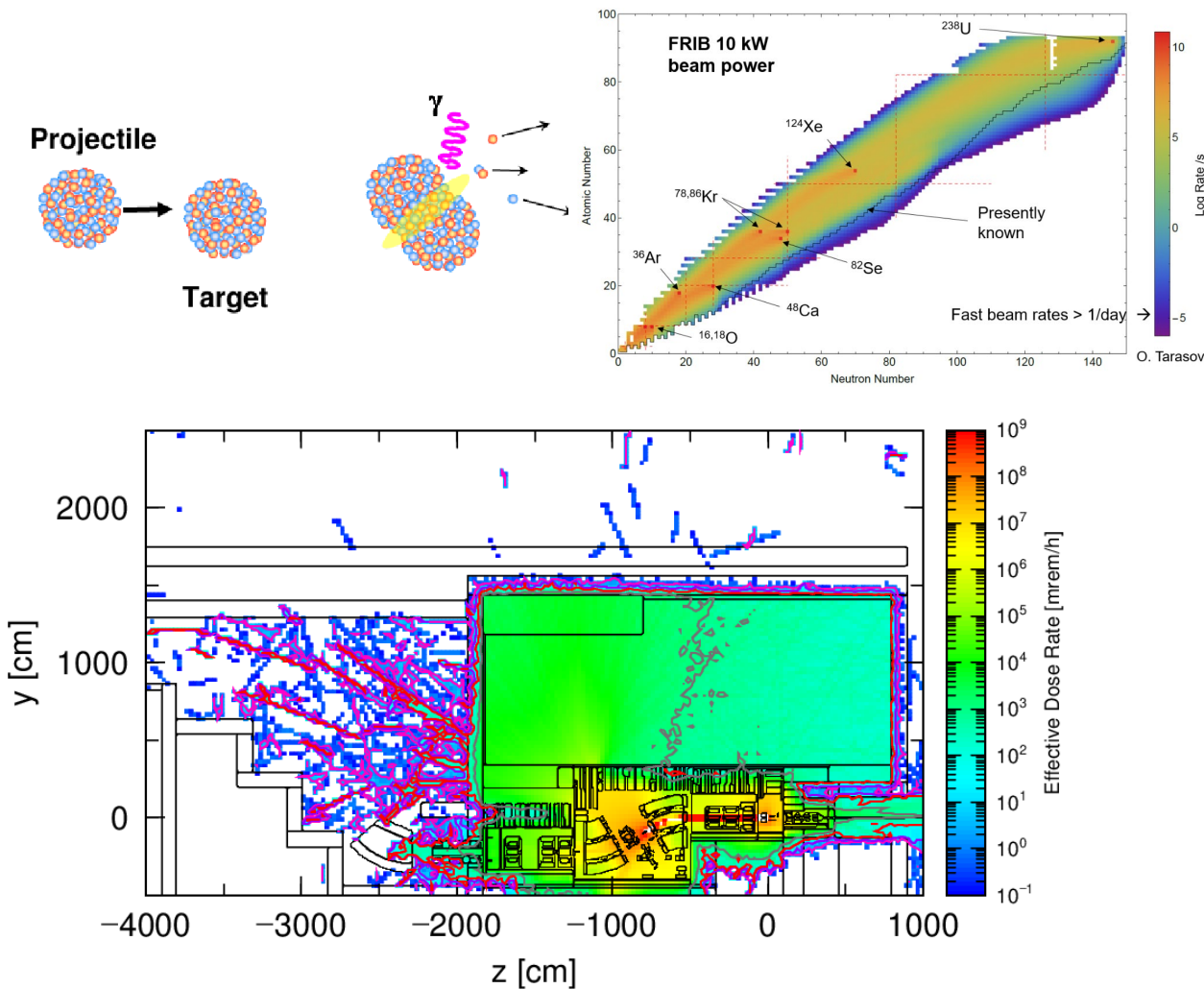


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ENERGY

Office of
Science

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Secondary Radiation at FRIB



FRIB: Facility for Rare Isotope Beams

- Multiple radiation scenarios (T. Ginter's talk)
- Accelerator facility: primary beams from ^1H to ^{238}U
- Max. Energy: ~ 200 MeV/u (upgrade 400 MeV/u)
- Beam power up to 400 kW
- Strong neutron fields

Systematic Uncertainties in Transport Models

Simulation \longleftrightarrow Reality

Geometry

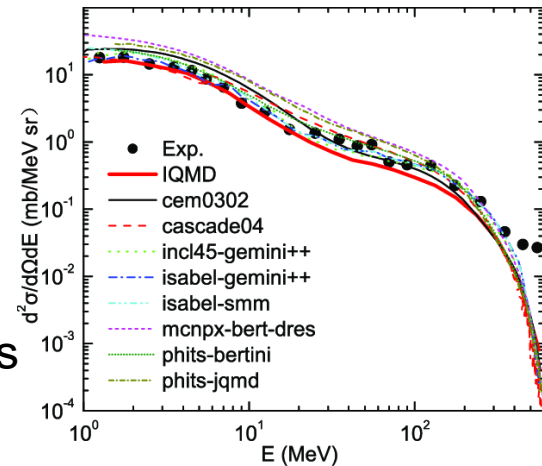
- Overlapping, gaps, shape
- Material composition, density
- Non-uniformities, shield cracks

Nuclear Data

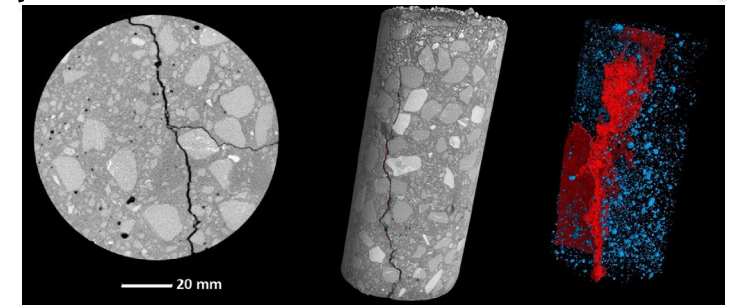
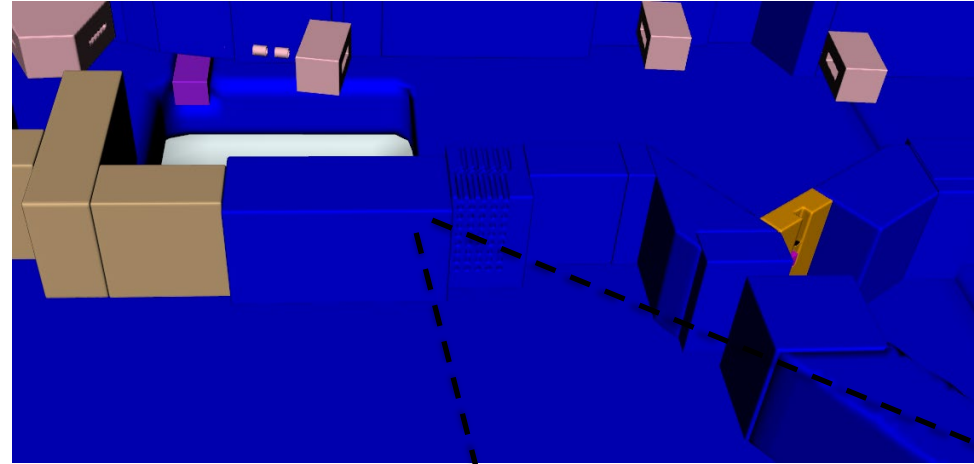
- Cross sections
- Systematics
- Reaction probes

Models

- Physics assumptions
- Approximations



Xu, Phys. Rev. C., **101**, 024609 (2020)



Plessis, Constr. Build. Mat., **199**, 637 (2019)

Bayesian Inference

A consistent approach that makes use of a prior probability (hypothesis) and data (evidence) to estimate a posterior probability

$P(\theta)$: prior probability

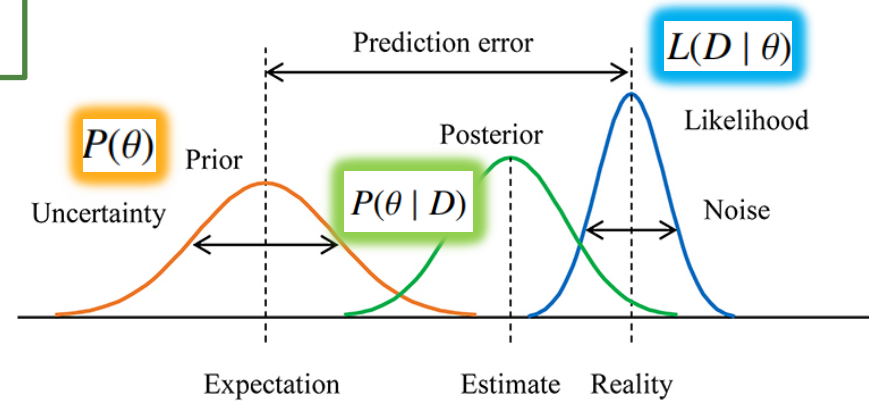
$P(\theta | D)$: posterior probability

$L(D | \theta)$: likelihood function

θ : parameters

D : data

Z : normalization

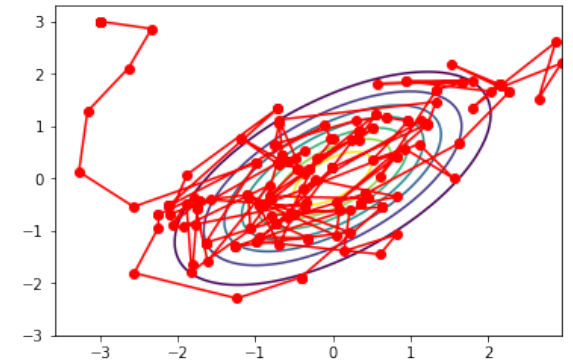


Bayes' Rule

$$P(\theta | D) = \frac{L(D | \theta)P(\theta)}{Z}$$

Markov chain Monte Carlo

Stochastic sampling the posterior probability using a random walk



Drawback: requires a large number of evaluations

Yanagisawa, Front. Comput. Neurosci., **13** (2019)

Surrogate Modeling: Machine Learning

It is necessary to use a surrogate model that emulates or mimics the radiation transport results while significantly reducing the computational time required for each sampling evaluation.

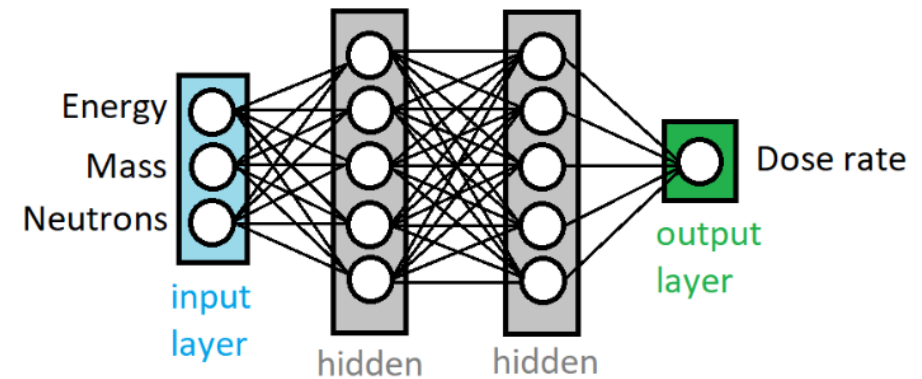
MC Radiation Transport (~ hours)



ML Surrogate Model (~ seconds)



Multidimensional regression problem



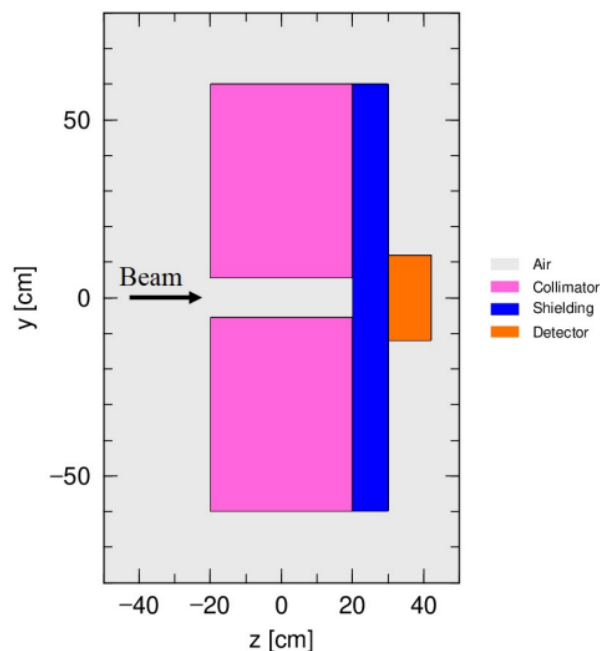
See R. Pal Chowdhury's poster

Non-intrusive approach

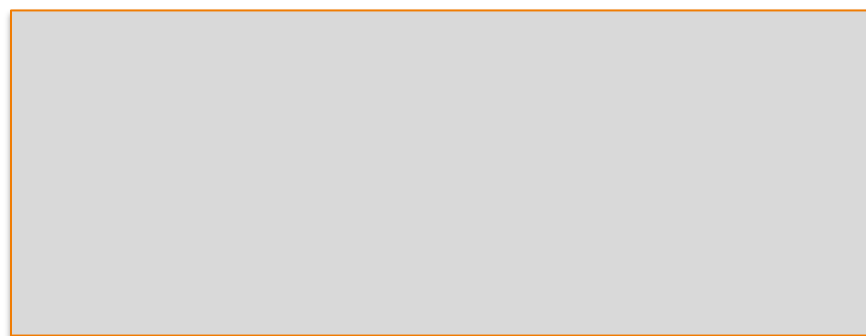
Zamora et. al., Proceedings SATIF-15, (2021)

Application 1 Neutron Shield

Radiation Transport Simulation Using PHITS



Convolutional Neural Network



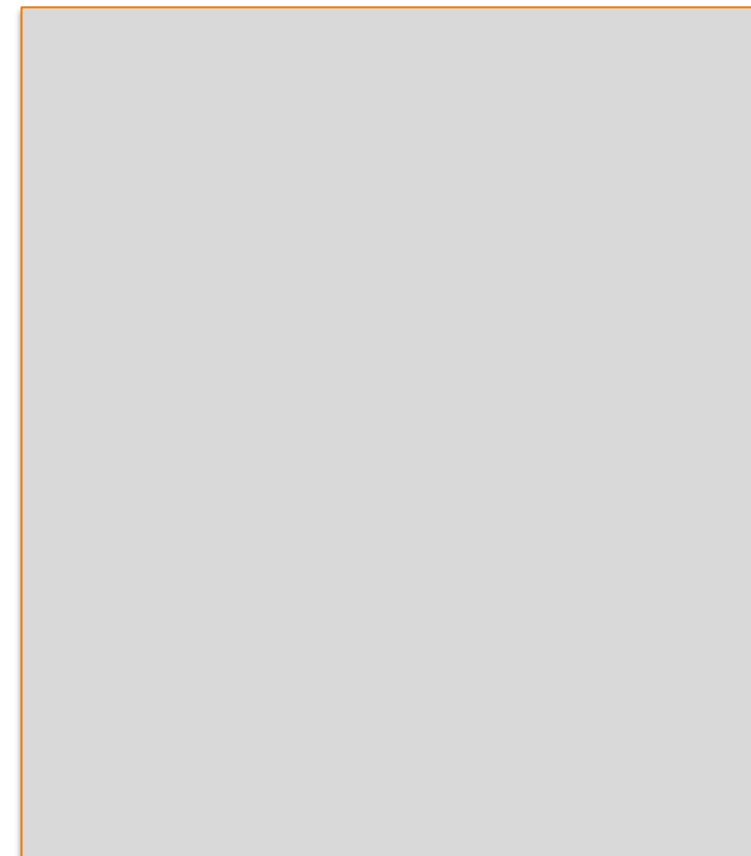
Nakao et al., Nucl. Sci. Eng. ,**124**, 228 (1996)

**Transmission Through Shields of Quasi-Monoenergetic
Neutrons Generated by 43- and 68-MeV Protons – I:
Concrete Shielding Experiment and Calculation
for Practical Application**

Noriaki Nakao*

Tohoku University, Cyclotron and Radioisotope Center, Aramaki, Aoba-ku, Sendai 980, Japan

Predicted Neutron Fluence



Zamora et. al., in preparation

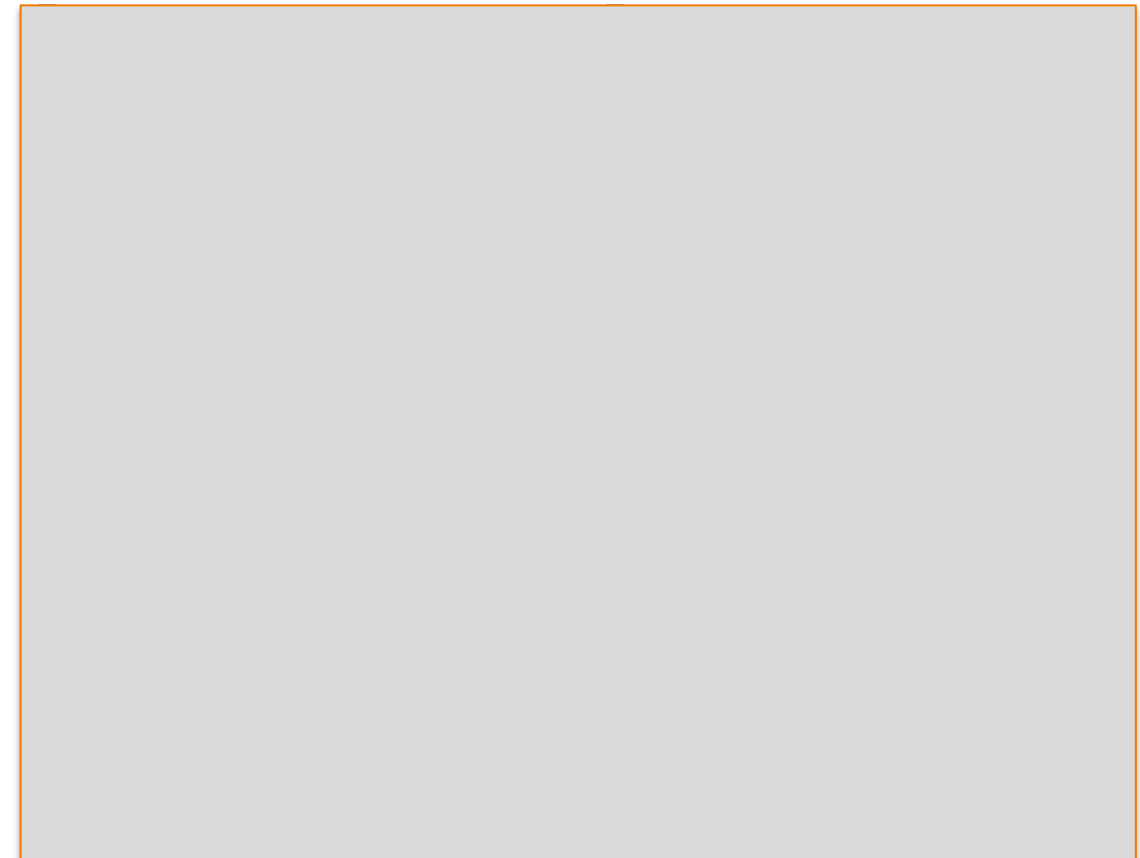
Uncertainty Quantification for Neutron Shield

Posterior Probability Density Functions



Neutron Fluence

Data: Nakao et al., Nucl. Sci. Eng. ,**124**, 228 (1996)



Cumulative Probability

Nominal Thickness	3%	16%	50%	84%	97%
25	23.1	24.0	25.1	26.2	27.2
50	49.2	49.8	50.7	51.5	52.4
100	98.1	98.8	99.8	100.7	101.6
150	148.4	149.3	150.4	151.5	152.4

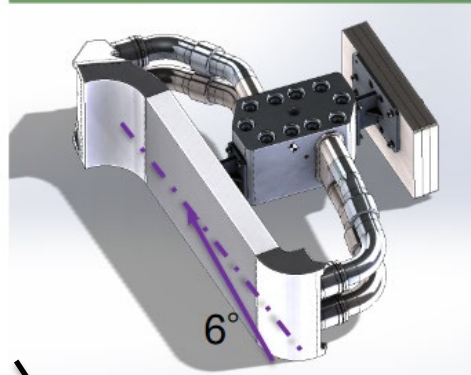
Zamora et. al., in preparation



Application 2

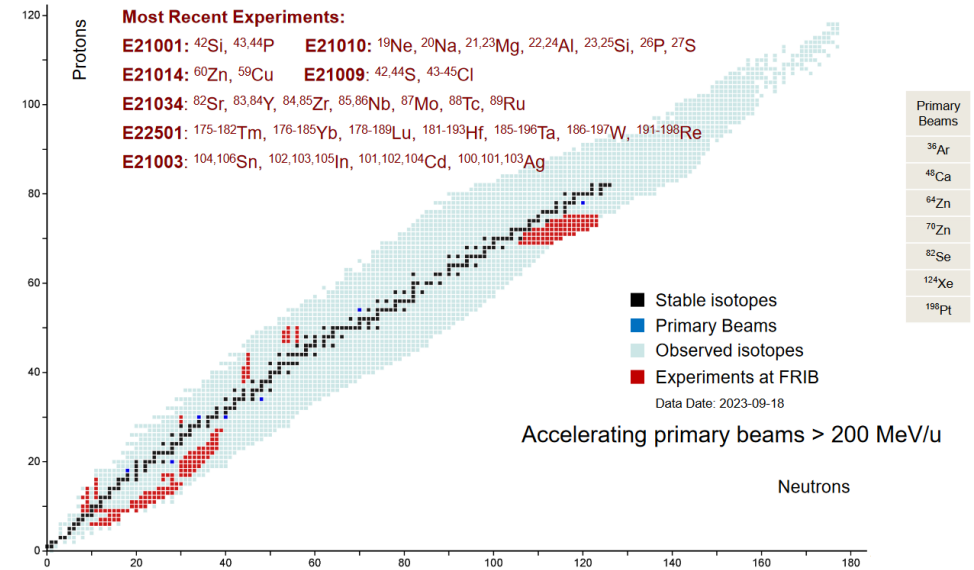
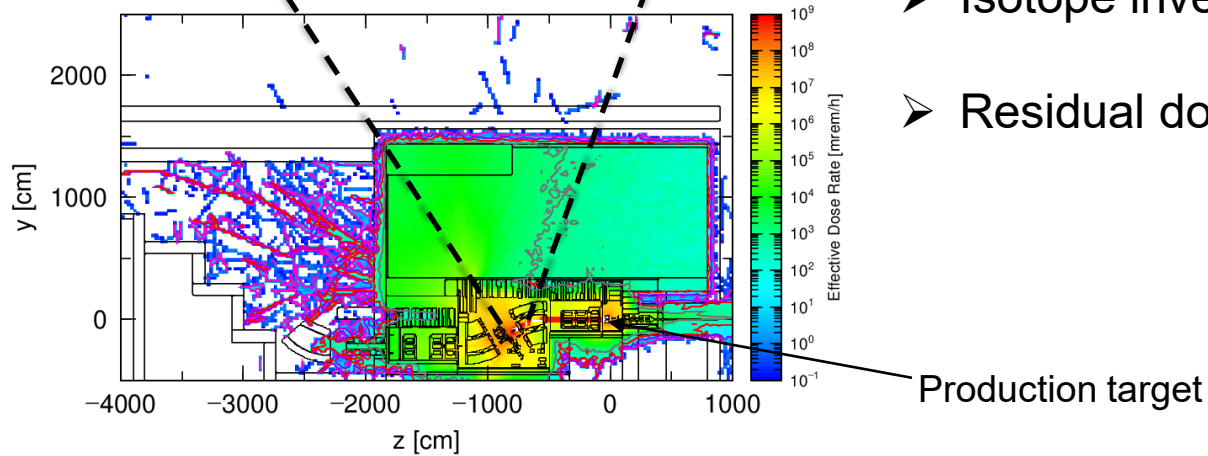
Beam Dump Activation

S-Shape Beam Dump



PHITS + DCHAIN Calculations

- 19 primary beams for PAC1: energy, power, element
- Uncertainty propagation
- Activation
- Isotope inventory
- Residual dose rate

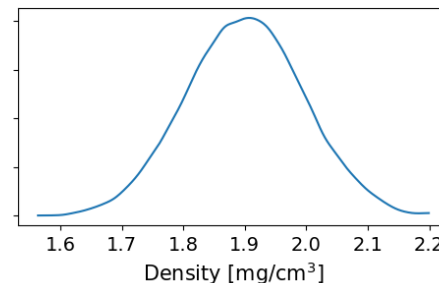
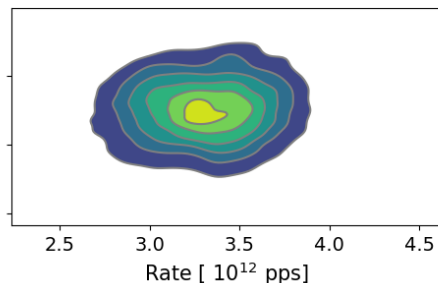
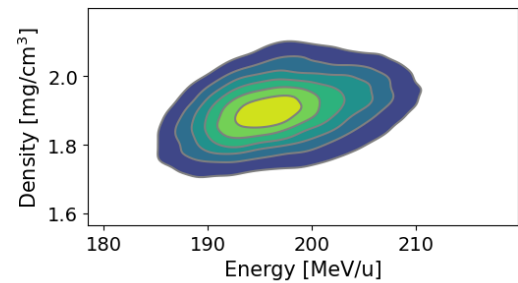
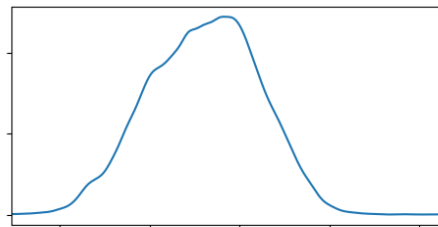
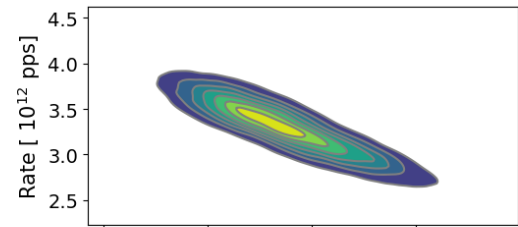
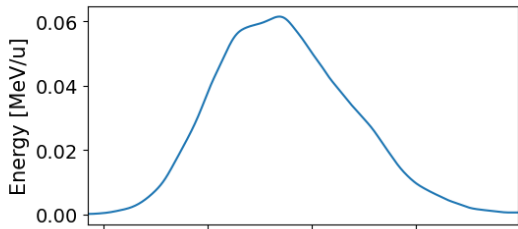


Application 2

Posterior Probability Distributions

Parameters:

- Density of the production target
- Beam energy after the target
- Beam rate



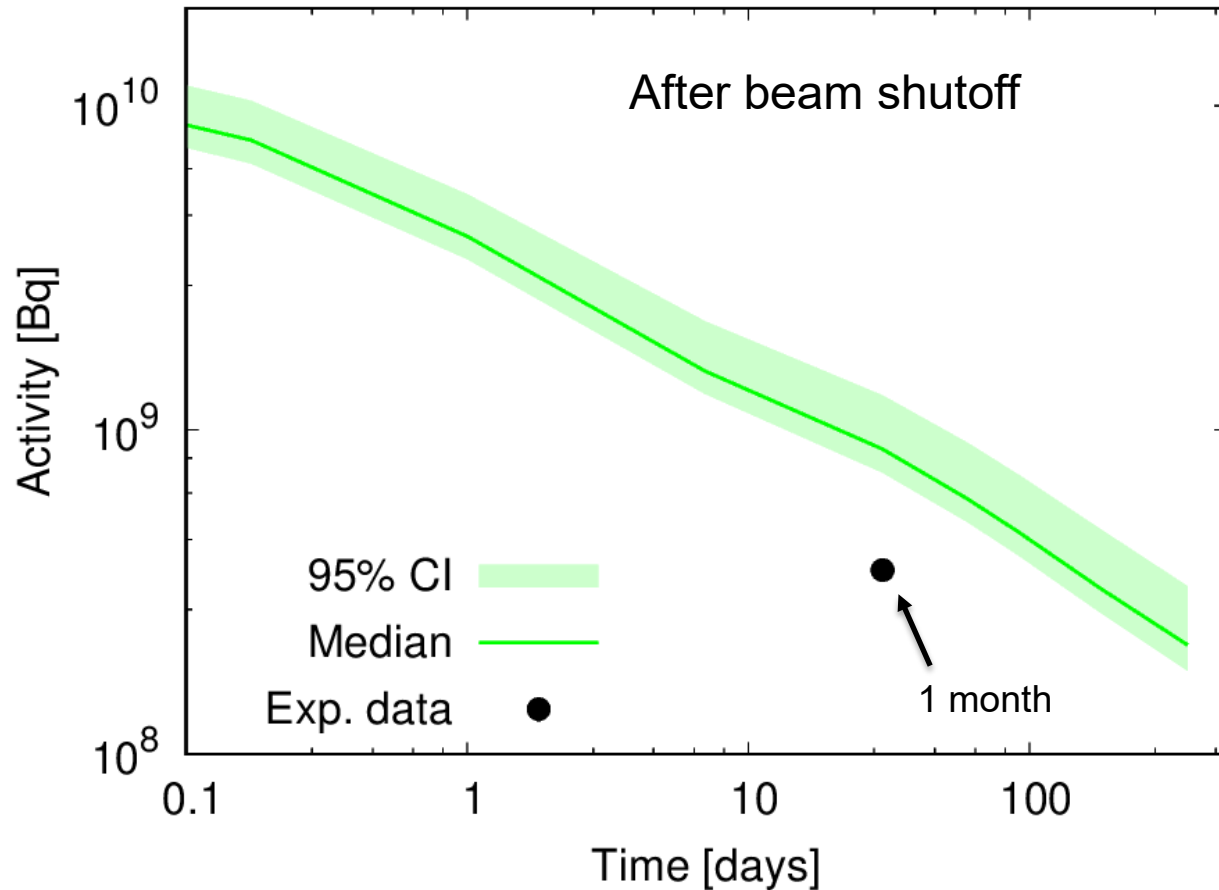
Propagation of systematic uncertainty through DCHAIN (burnup code). UQ tested with synthetic data (std = 10%)

It is possible to study correlations and parameter sensitivity

Application 2

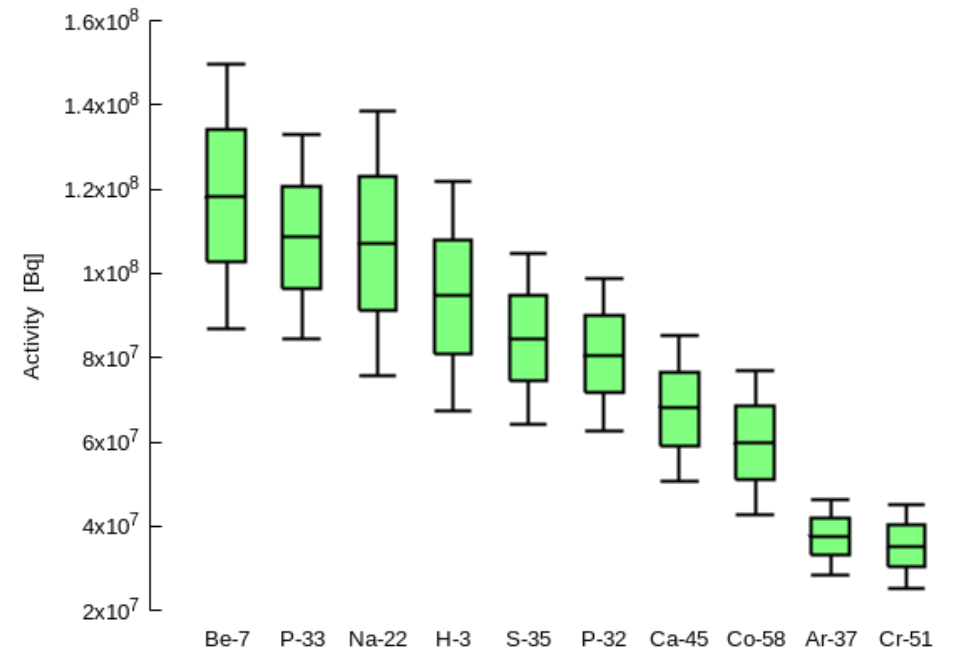
Activation After One-Year of FRIB Operation

1 year operation (19 experiments) ~ 5 kW



Uncertainty in the nuclide inventory.
Isotope by isotope basis

Top 10 isotopes after 1 month cooling



Summary

- A Bayesian inference framework was developed to quantify uncertainties arising from radiation transport calculations. The approach is non-intrusive and can be utilized for various codes.
- Machine learning application is employed for surrogate modeling. The model emulates the radiation transport results while significantly reducing the computational time required for each sampling evaluation.
- The method can be used to study systematic uncertainty propagation and parameter sensitivity involving coupling to burnup codes.



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- The method can be used to study systematic uncertainty propagation and parameter sensitivity involving coupling to burnup codes.

Thanks for your attention!

