



Multifaceted, coded nuclear data libraries assemblage: TENDL-2023

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The making of TENDL-2023 – repeatability and innovation

7 incident particles: alpha, gamma, deuteron, proton, helium, triton and neutron induced libraries

2850 targets Z=1-115, Hydrogen to Moscovium, **including as target some 519 m (1st), 29 n (2nd) isomeric states ($T_{1/2} > 1s$)**

TENDL describes all open reaction channels, product yields, emitted spectra, and short-lived daughter radionuclides ($T_{1/2} > 0.1s$) up to 200 MeV, with covariance derived from **reference input parameters variation**

12th version

Group #	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18
Period																		
1	1 H																	2 He
2	3 Li	4 Be											5 B	6 C	7 N	8 O	9 F	10 Ne
3	11 Na	12 Mg											13 Al	14 Si	15 P	16 S	17 Cl	18 Ar
4	19 K	20 Ca	21 Sc	22 Ti	23 V	24 Cr	25 Mn	26 Fe	27 Co	28 Ni	29 Cu	30 Zn	31 Ga	32 Ge	33 As	34 Se	35 Br	36 Kr
5	37 Rb	38 Sr	39 Y	40 Zr	41 Nb	42 Mo	43 Tc	44 Ru	45 Rh	46 Pd	47 Ag	48 Cd	49 In	50 Sn	51 Sb	52 Te	53 I	54 Xe
6	55 Cs	56 Ba	*	72 Hf	73 Ta	74 W	75 Re	76 Os	77 Ir	78 Pt	79 Au	80 Hg	81 Tl	82 Pb	83 Bi	84 Po	85 At	86 Rn
7	87 Fr	88 Ra	**	104 Rf	105 Db	106 Sg	107 Bh	108 Hs	109 Mt	110 Ds	111 Rg	112 Cn	113 Nh	114 Fl	115 Mc	116 Lv	117 Uus	118 Uuo
* Lanthanoids	57 La	58 Ce	59 Pr	60 Nd	61 Pm	62 Sm	63 Eu	64 Gd	65 Tb	66 Dy	67 Ho	68 Er	69 Tm	70 Yb	71 Lu			
** Actinoids	89 Ac	90 Th	91 Pa	92 U	93 Np	94 Pu	95 Am	96 Cm	97 Bk	98 Cf	99 Es	100 Fm	101 Md	102 No	103 Lr			

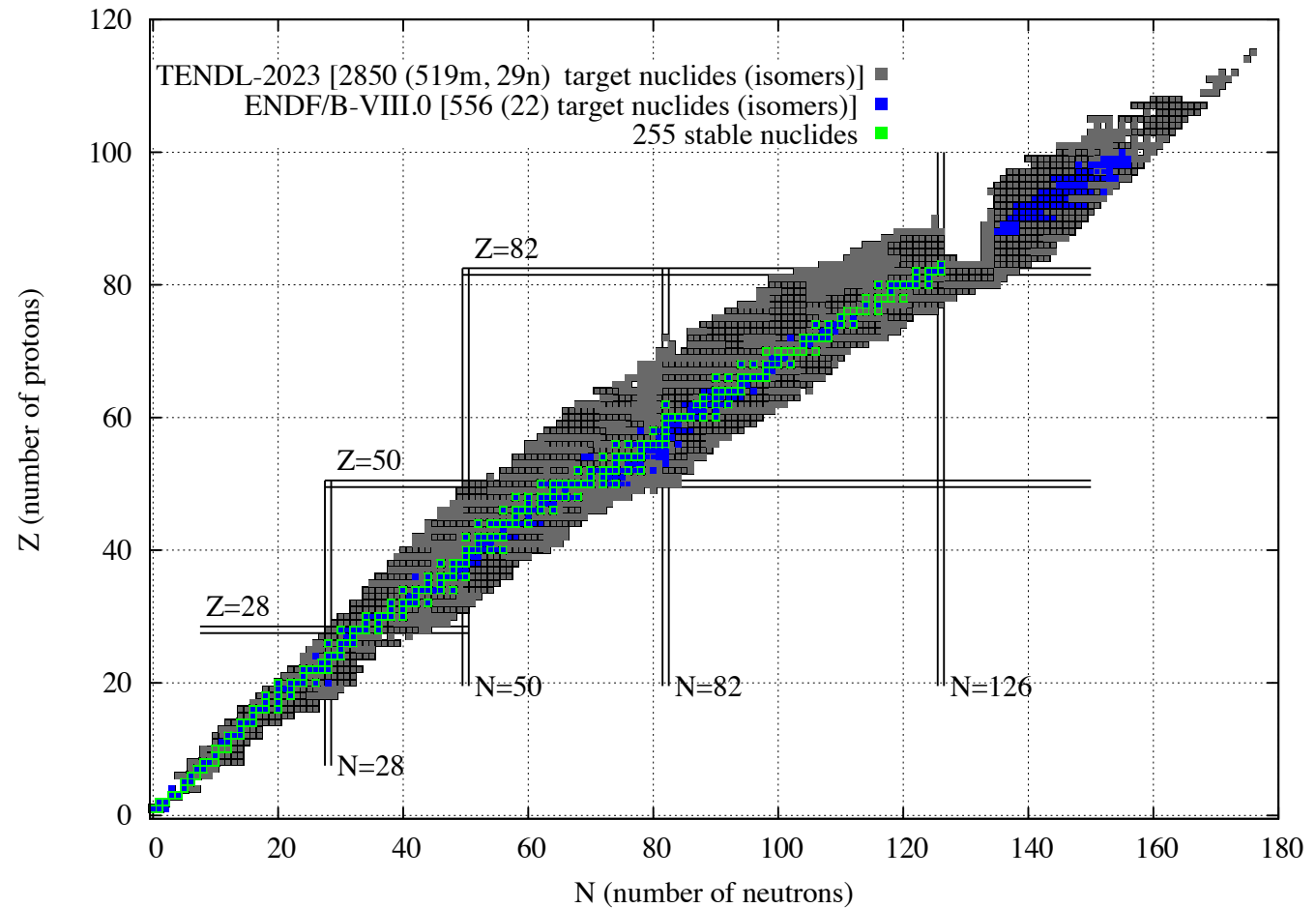
Nuclear landscape: Isotopic targets: 2850

7 incident particles

- neutron*
- gamma
- proton
- deuteron
- triton
- alpha
- helion

*complete for transport,
activation-transmutation-
material sciences in term of
MF's, targets and
daughters

795 for n-JENDL-5
816 for EAF-2010
~550 for n-ENDF/B, JEFF,
and CENDL



Nuclear landscape: Isotopic residuals & decays

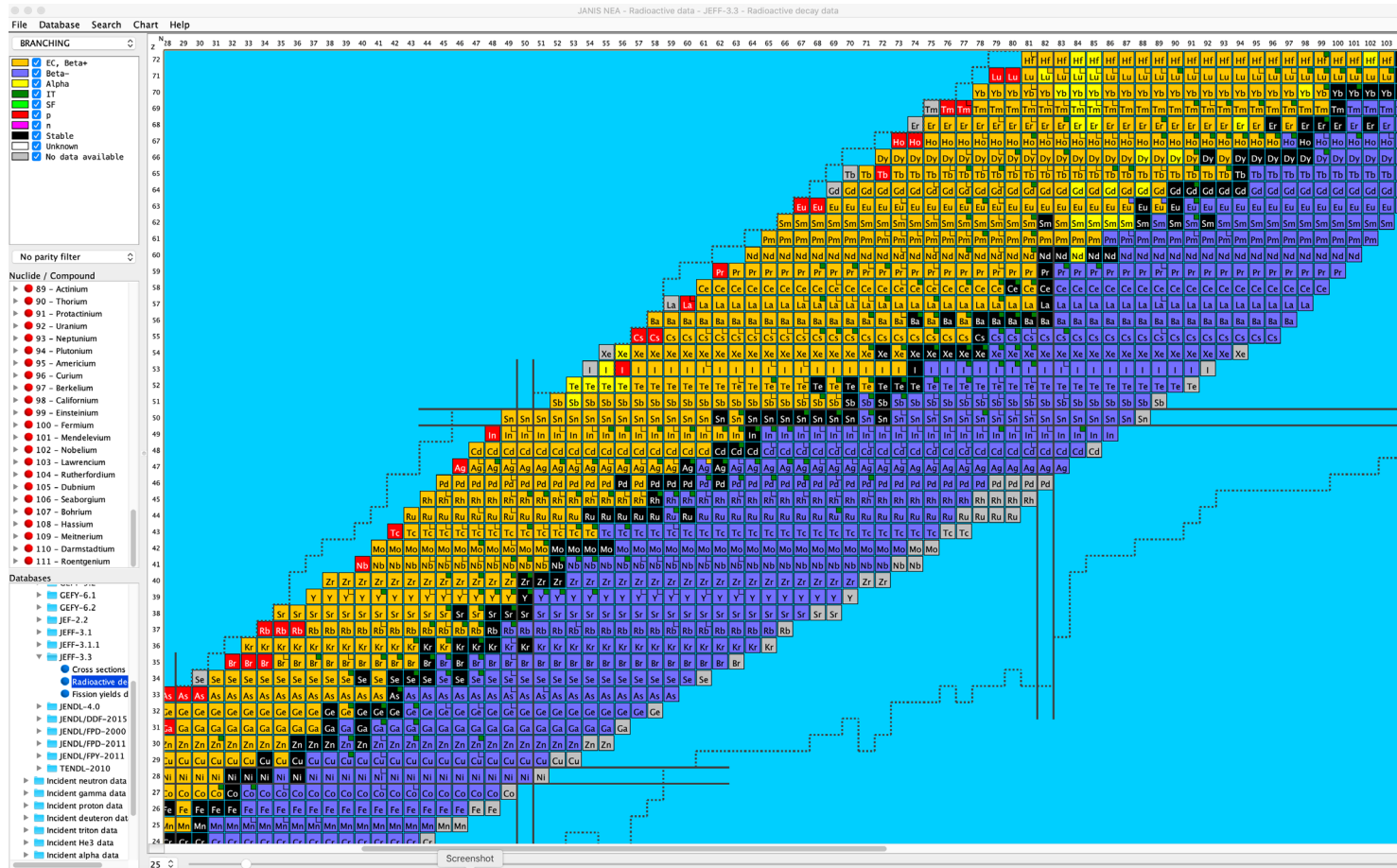
Radioactive decay products: 4035 “reaction daughters” or “residuals”

Ground and isomeric daughters:
($T_{1/2} > 0.1s$)

Decay data files are not a part of TENDL

but

MF-8 contains the metastable Elevel from RIPL-4



Portal https://tendl.web.psi.ch/tendl_2023/tendl2023.html

ENDF-6: explicit s30 GNDS Special: 600 MeV Tabular forms Astrophysics

TENDL-astro 2023

Cross sections, reaction rates and MACS for astrophysics

Recommended quantities

Cross sections

1. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)
2. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)
3. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)

Reaction rates

1. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)
2. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)
3. [\(g,n\)](#), [\(g,p\)](#), [\(n,n\)](#)

Normalized partition function [GFD](#)

30 keV Laboratory Maxwellian Averaged (n,g) Cross Sections [MACS](#)

Description of TALYS models

Below are various links to 8892 isotopes for astrophysics applications (cross sections, reaction rates and MACS) based on TALYS calculations (version 1.96).

Different reaction "model sets" were used: "a model set" represents a combination of 9 TALYS models:

1. Gamma strength function (values 8 or 9): either Gogny DIM HFB+QRPA, or SML0
2. Level density (values 1, 2 or 5): Constant temperature + Fermi gas model, or Back-shifted Fermi gas model, or Microscopic level densities (Skyrme force)
3. JLM microscopic optical model potential or KD optical model (values y or n)
4. Gamma strength function for M1 (values 3 or 8): Harries-Pock RCS tables or Gogny DIM HFB+QRPA
5. Collective enhancement (values y or n): yes or no
6. Width fluctuation (values 0, 1 or 2): Moldauer model, or Hofmann-Richter-Iepel-Waldmueller model
7. Mass model (values 0, 1, 2 or 3): Duflo-Zuker formula, Moeftel table, Gogny HFB-Skyrme table, or HFB-Gogny DIM table (except for known masses, v used)
8. Alpha optical model (values 5 or 6): Demetresou-Gerilyou, or Avrigeanu
9. Fission model (values 1 or 5): "experimental" fission barriers, or WKB approximation for fission path model.

Each of the set models is named with 9 values, such as "91n3n1261" (default TALYS model), or 8568n1261. These values correspond to the ones in the [TALYS](#).

Application libraries & tar

Last update: 21 December 2023

1. Neutron

- 2850 ENDF [files](#) (s30 files, 2.9 Gb, with H1, H2, H3, He3, He4, Li6, Li7, B10, B11, Be7, Be9, C12, C13, N14, N15, O16, O17, O18, F19, Th232, U233, U235, U238 and Pu239 from ENDF/B-VIII.0)
- 2864 special 600 MeV ENDF [files](#) (3.9 Gb),
- 2841 special s60 ENDF [files](#) (6.1 Gb),

2. Proton

- 2855 ENDF [files](#) (s30 files, 2.0 Gb with H1, H2, H3, He3, He4, Li6, Li7, Be9, C12, C13, N14 and O16 from ENDF/B-VIII.0),
- 2850 special s0 ENDF [files](#) (1.1 Gb),
- 2850 special 600 MeV ENDF [files](#) (5.3 Gb),

3. Deuteron

- 2850 ENDF [files](#) (s30 files, 2.7 Gb with H2, H3, He3, Li6, Li7 from ENDF/B-VIII.0),
- 2860 special s0 ENDF [files](#) (1.3 Gb),
- 2846 special 600 MeV ENDF [files](#) (7.5 Gb),

4. Triton

- 2865 ENDF [files](#) (s30 files, 2.3 Gb, with H3, He3 and Li6 from ENDF/B-VIII.0),
- 2860 special s0 ENDF [files](#) (1.3 Gb),
- 2850 special 600 MeV ENDF [files](#) (6.7 Gb),

5. He3

- 2821 ENDF [files](#) (s30 files, 2.4 Gb, with He3 and Li6 from ENDF/B-VIII.0),
- 2820 special s0 ENDF [files](#) (1.2 Gb),
- 2848 special 600 MeV ENDF [files](#) (6.2 Gb),

6. Alpha

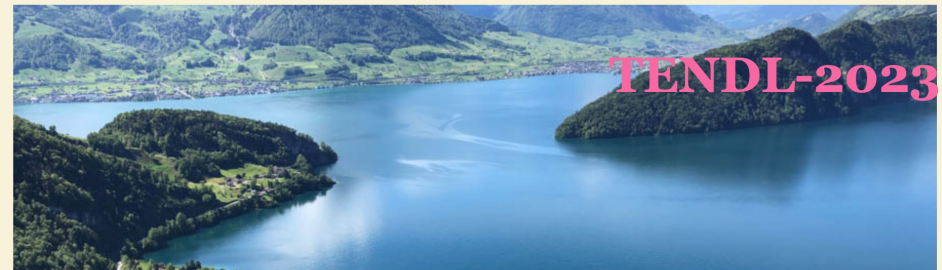
- 2835 ENDF [files](#) (s30 files, 1.4 Gb),
- 2855 special s0 ENDF [files](#) (1.0 Gb),
- 2850 special 600 MeV ENDF [files](#) (2.0 Gb),

7. Gamma

- 2825 ENDF [files](#) (s30 files, 1.4 Gb, with H2 from ENDF/B-VIII.0),
- 2864 special s0 ENDF [files](#) (1.0 Gb),
- 2526 special 600 MeV ENDF [files](#) (3.3 Gb),

TALYS-based evaluated nuclear data library

Home Reference & us Citations Feedback TALYS



“ We believe that our great goal can be achieved with systematism and reproducibility. We are so outside the box, that the box is a point”

How to reference

Sub-library files

1. Neutron
2. Proton
3. Deuteron
4. Triton
5. He3
6. Alpha
7. Gamma
8. Fission yields
9. Thermal scattering
10. For astrophysics

Application libraries & tar files (ENDF, GND, ACE, PENDE...)

V&V

Total Monte Carlo files

3. Random ENDF-6 files from other libraries
4. Random ACE files based on ENDF/B-VII.1

TENDL-2023: (release date: December 22, 2023)

Last update: December 21, 2023

TENDL is a nuclear data library which provides the output of the TALYS nuclear model code system for direct use in both basic physics and applications. The 12th version is TENDL-2023, which is based on both default and adjusted TALYS calculations and data from other sources (previous releases can be found here: [2008](#), [2009](#), [2010](#), [2011](#), [2012](#), [2013](#), [2014](#), [2015](#), [2017](#), [2019](#) and [2021](#)).

Up to 2014, TENDL was produced at NRG Petten. Since 2015, TENDL is mainly developed at PSI and the IAEA (Nuclear Data Section). Still, many people contribute to TENDL with the testing and processing of the files.

TENDL contains evaluations for seven types of incident particles, for all isotopes living longer than 1 second: Z=1 ¹H to Z=115 ²⁹¹Mc (about 2850 isotopes), up to 200 MeV, with covariances (test files up to 600 MeV are also provided).

TENDL is **not** a default or shadow library. Not a single neutron evaluation is based on default calculations. With the HFR approach, all resonances follow statistical hypothesis. For major isotopes, greater care was used during the evaluation process.

All TENDL-2023 neutron files are original except 24. The 24 following files are taken from ENDF/B-VIII.0 for neutrons: ^{1,2,3}H, ^{3,4}He, ^{6,7}Li, ^{10,11}B, ^{7,9}Be, ^{12,13}C, ^{14,15}N, ^{16,17,18}O, ¹⁹F, ²³²Th, ^{233,235,238}U and ²³⁹Pu.

A set of tools, called T6, was used to produce it. T6 stands for TALYS, TEFAL, TASMAN, TARES, TAFIS and TANES. Each code produces a part of the library. Processing tools such as NJOY, CALENDF, PREPRO are also used in T6. These codes, and the processing steps are developed by A.J. Koning, D. Rochman and J.Ch. Sublet. Still, the help and feedback of the whole nuclear data, processing and user community is extremely useful. TENDL would not exist without the constructive remarks from all over the world.

Portal <https://tendl.web.psi.ch/tendl2023/tendl2023.html>

TENDL-2023 Nuclear data library

Neutron sub-library for C (Z=6) and A=11

Tabular forms

YANDF-0.2

Angle, xs direct and compound

Special: 600 MeV

6 ENDF forms

Processed plots

Tabulated data (fast neutron range)

1. [Tabular angular distributions](#) (En - angle - cross section)
2. [Tabular residual cross sections](#) (En - Residual product - cross section)
3. [Tabular total and partial cross sections](#) (En - cross section)

Evaluated formatted data (i.e. ENDF)

1. [The TENDL file](#)
2. Special [ENDF](#) file with MF12/MT102 (so-called s20 file)
3. Special [ENDF](#) file similar to the TENDL file, but with a transition to MT5 at 60 MeV (so-called s60 file)
4. Special [file](#) (= the TENDL file, but up to 600 MeV) ←
5. [EAF file](#) (European Activation File) and associated [covariances](#)
6. [ACF file](#) (Activation File)

Processed plots

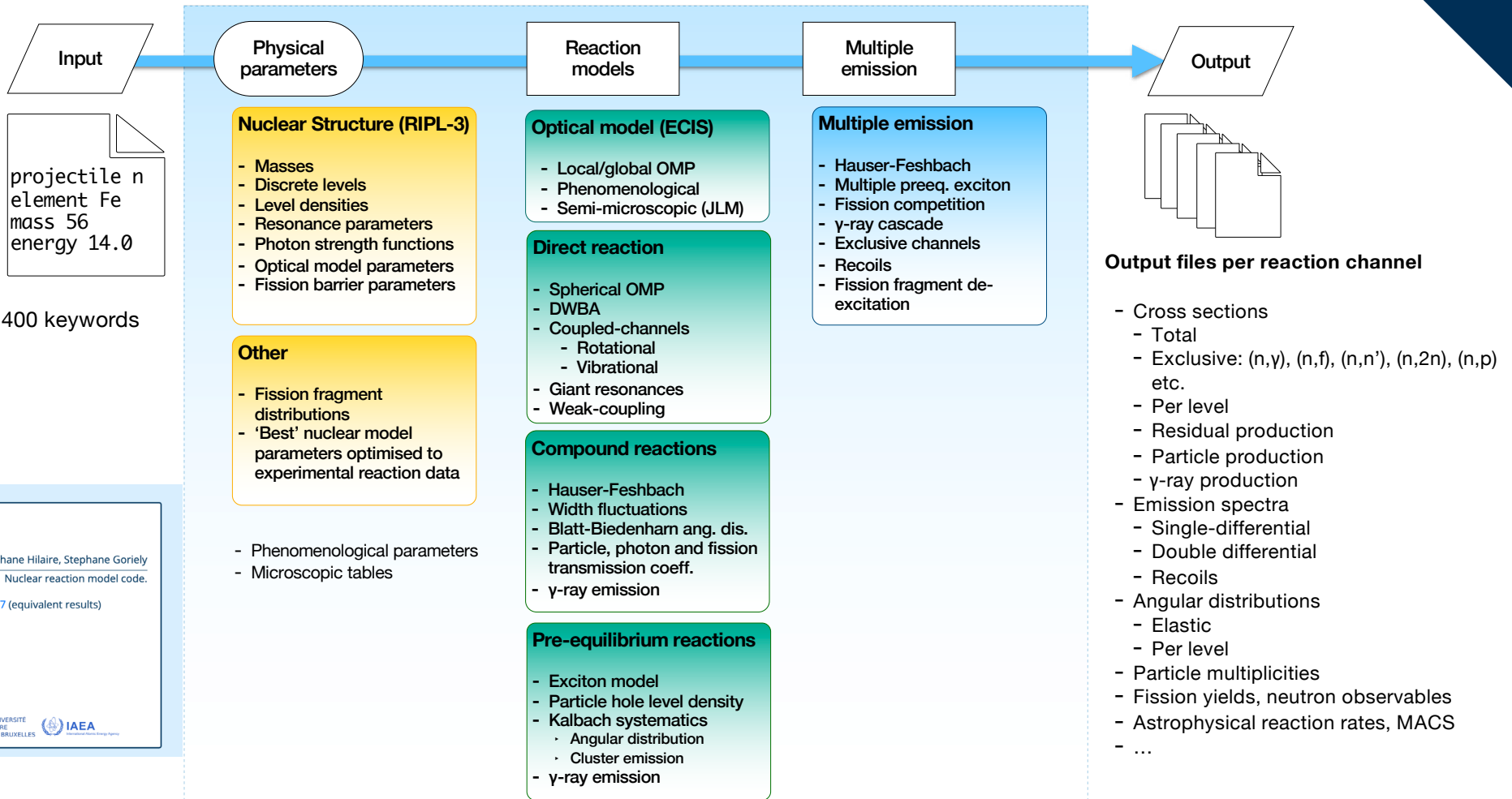
1. Plots of the cross sections and other quantities: [plot](#)
2. Plots from PREPRO: [plot](#)
3. Processed cross sections and covariances with NJOY in 187 groups: [matrix](#)

T6 input files

acf.inp	acf.inp_cov	eaf.inp	eaf.inp_cov	energies	gpf.inp	gpf.inp_cov	gpfs20.inp
gpfs20.inp_cov	gpfs60.inp	gpfs60.inp_cov	njoy.inp	talys.inp	talys.inp.0000	talys.inp.0001	talys.inp.0002
talys.inp.0003	talys.inp.9999	tasman.inp					

TALYS 2.0 & 1.97

<https://nds.iaea.org/talys/>



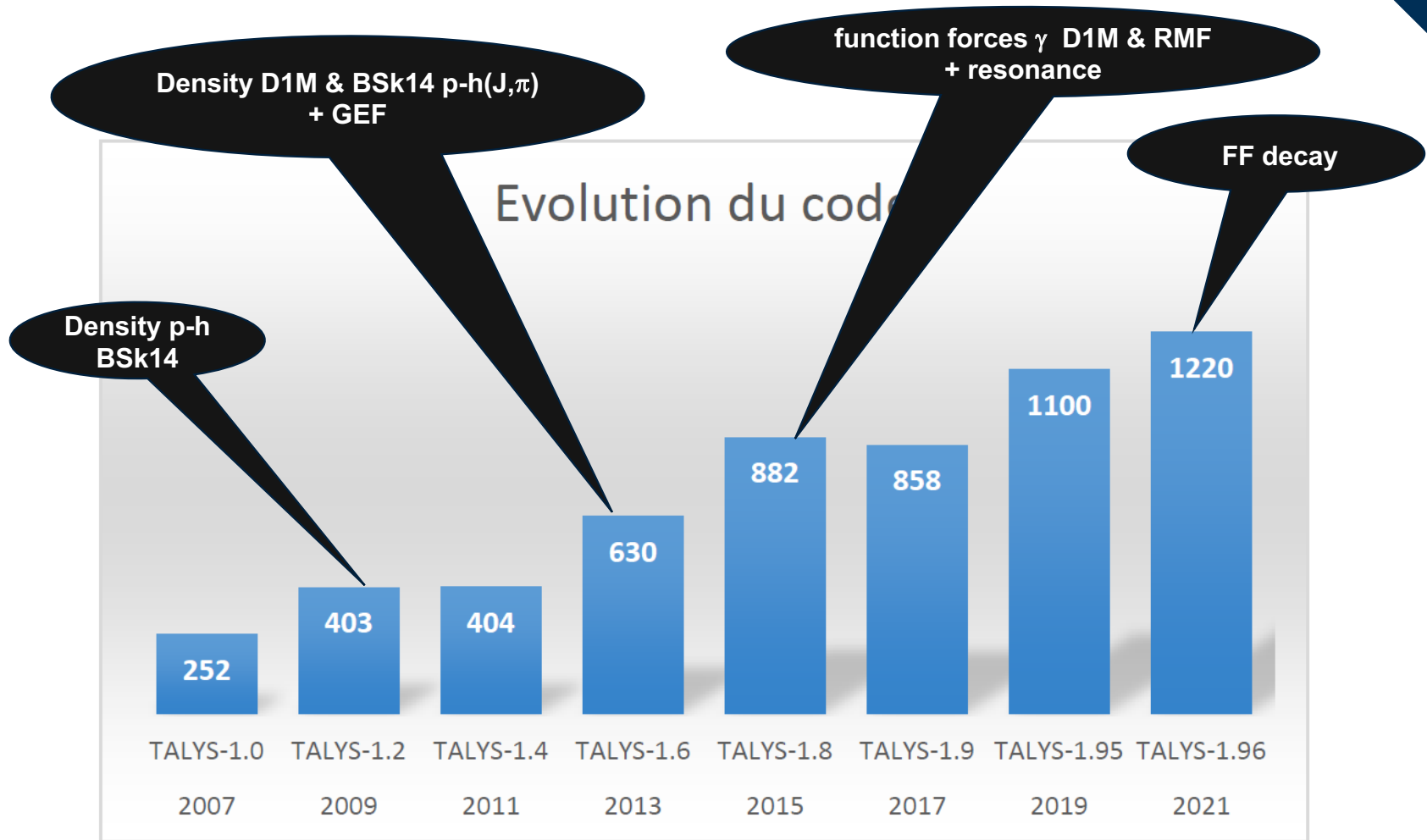
TALYS

Arjan Koning, Stephane Hilaire, Stephane Goriely
Nuclear reaction model code.

- Download TALYS-2.0 or TALYS-1.97 (equivalent results)
- Previous and Other Versions
- Read Tutorial
- Cite Reference
- Use TALYSworld
- Watch TALYS video lectures
- Access on GitHub

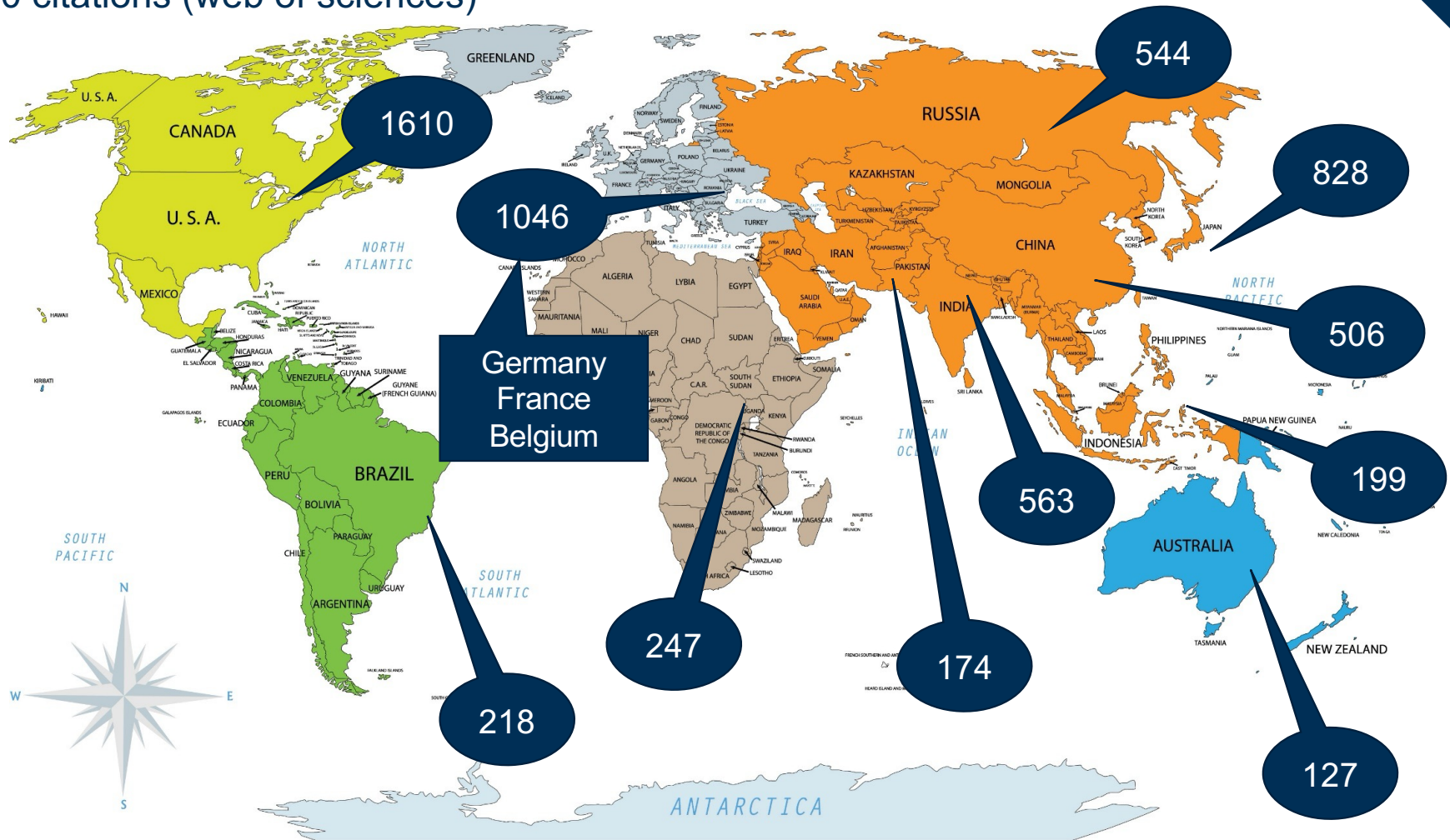
Created at    UNIVERSITE LIBRE DE BRUXELLES  IAEA

TALYS time steps evolution



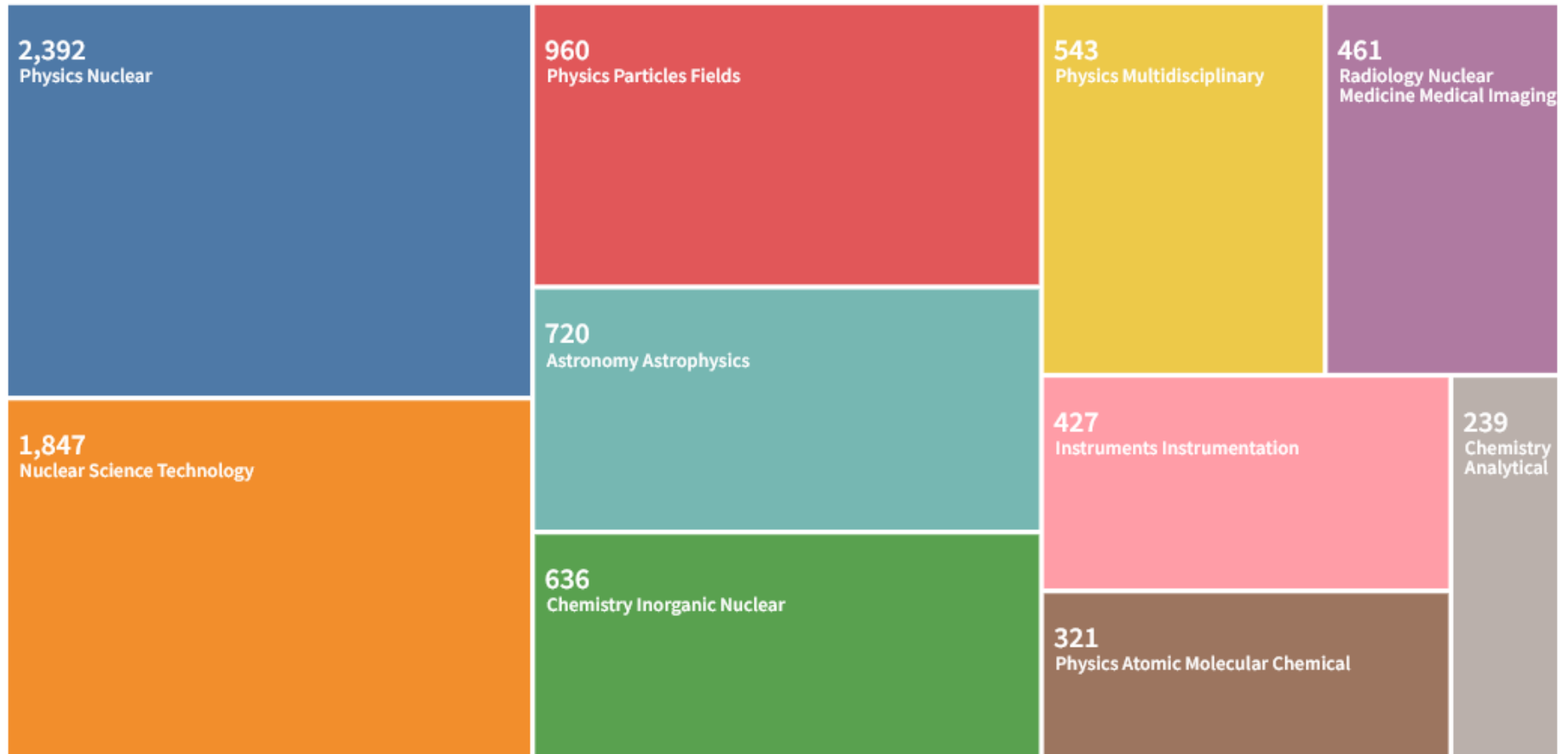
TALYS around the World

Around 5500 citations (web of sciences)



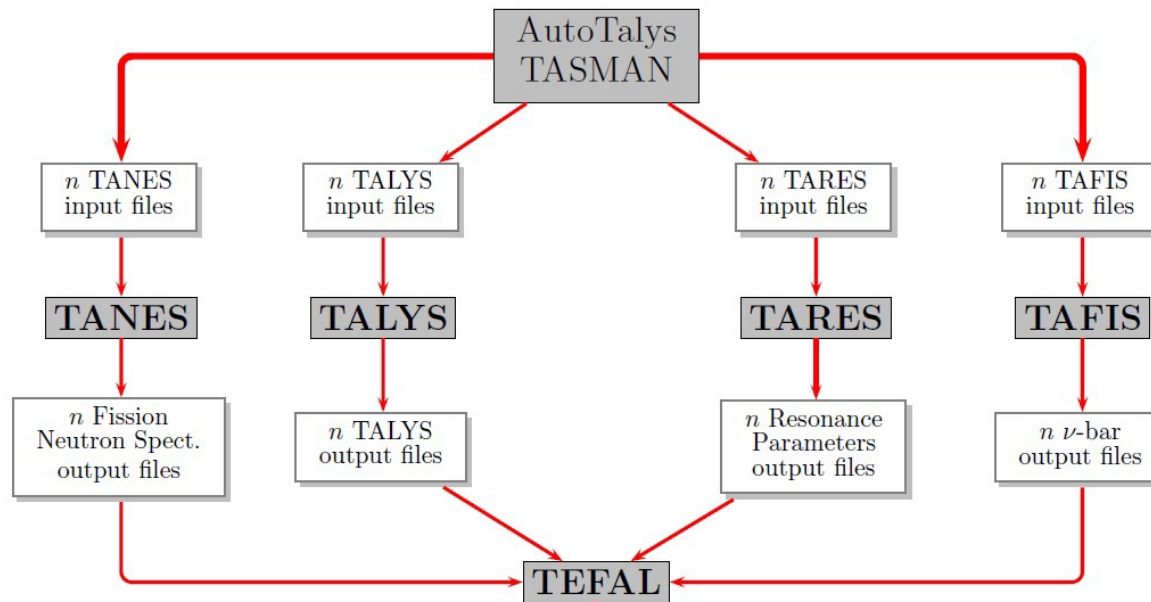
TALYS applications in the World

Around 5500 citations (web of sciences)



Previous T6: automatisisation – reproducibility - robustnes

TALYS, TANES, TARES, TAFIS, TEFAL, TASMAN



Default models input parameters

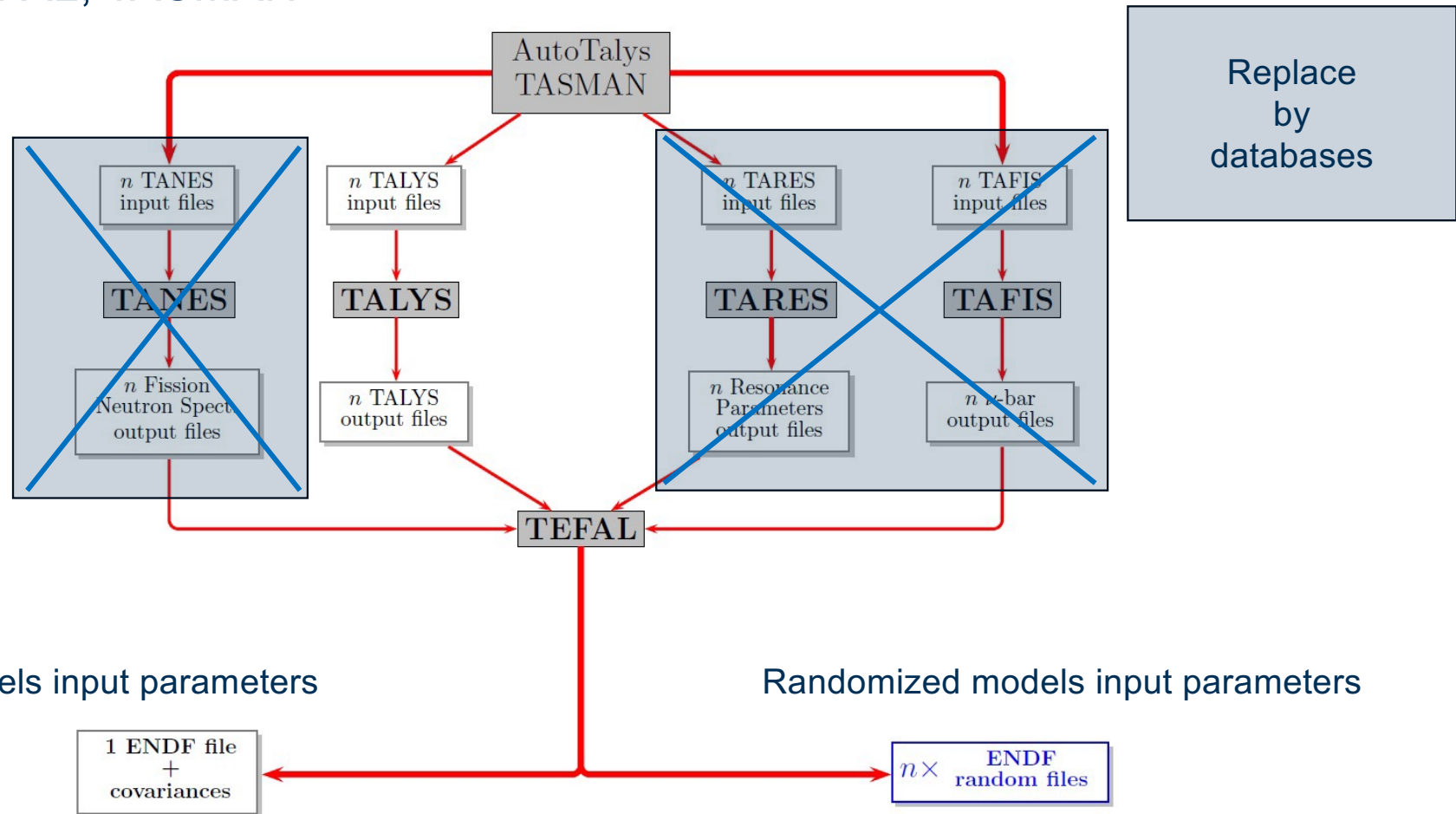
Randomized models input parameters

1 ENDF file
+
covariances

$n \times$ ENDF
random files

New T3: automatisisation – reproducibility - robustnes

TALYS, TEFAL, TASMAN



T3: TALYS, TEFAL, TASMAN + resbase

TALYS & Related Software

TALYS and the related packages are open source software and datasets ([MIT License](#)) for the simulation of nuclear reactions.

TALYS

Arjan Koning, Stephane Hilaire, Stephane Goriely

Nuclear reaction model code.

- Download TALYS-2.0 or TALYS-1.97 (equivalent results)
- Previous and Other Versions
- Read Tutorial
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- Use TALYSworld
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- Access on GitHub

Created at    UNIVERSITÉ LIBRE DE BRUXELLES  IAEA International Atomic Energy Agency

<https://nds.iaea.org/talys/>

TASMAN

Arjan Koning

Statistical software for TALYS: Uncertainties, sensitivities and optimization.

- Download TASMAN-2.0
- Read Tutorial
- Cite Reference
- Access on GitHub

TEFAL

Arjan Koning

Code to make ENDF-6 nuclear data libraries from TALYS.

- Download TEFAL-2.0
- Read Tutorial
- Cite Reference
- Access on GitHub

RESONANCETABLES

Arjan Koning, Dimitri Rochman

Database for thermal cross sections, MACS and average resonance parameters.

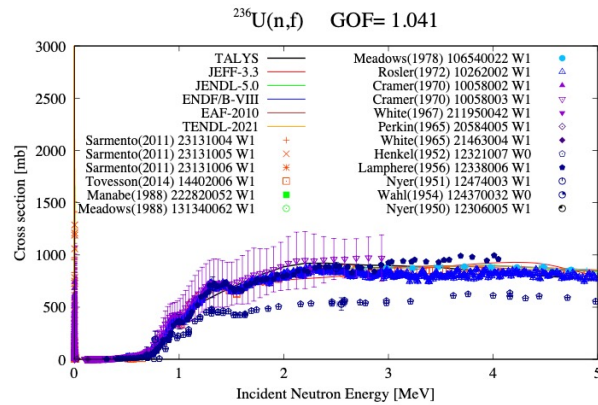
- Download RESONANCETABLES-2.0
- Read Tutorial
- Cite Reference
- Access on GitHub

Created at  IAEA International Atomic Energy Agency  PAUL SCHERRER INSTITUT

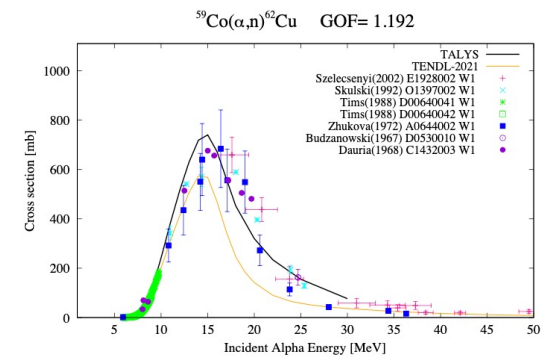
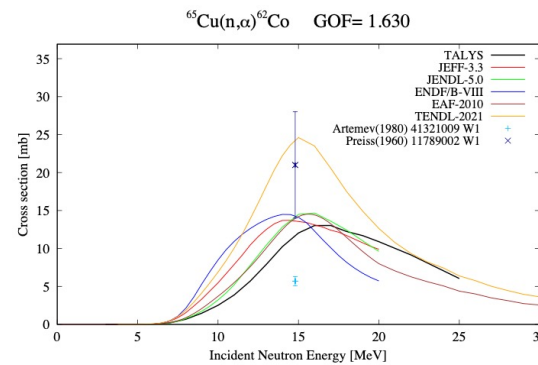
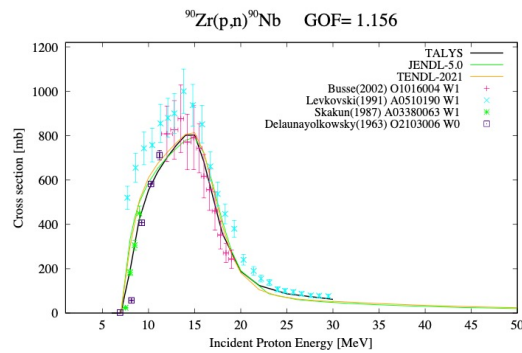
TALYS coded input parameters & GOF

Optimized parameters

Weight allocation on exp.



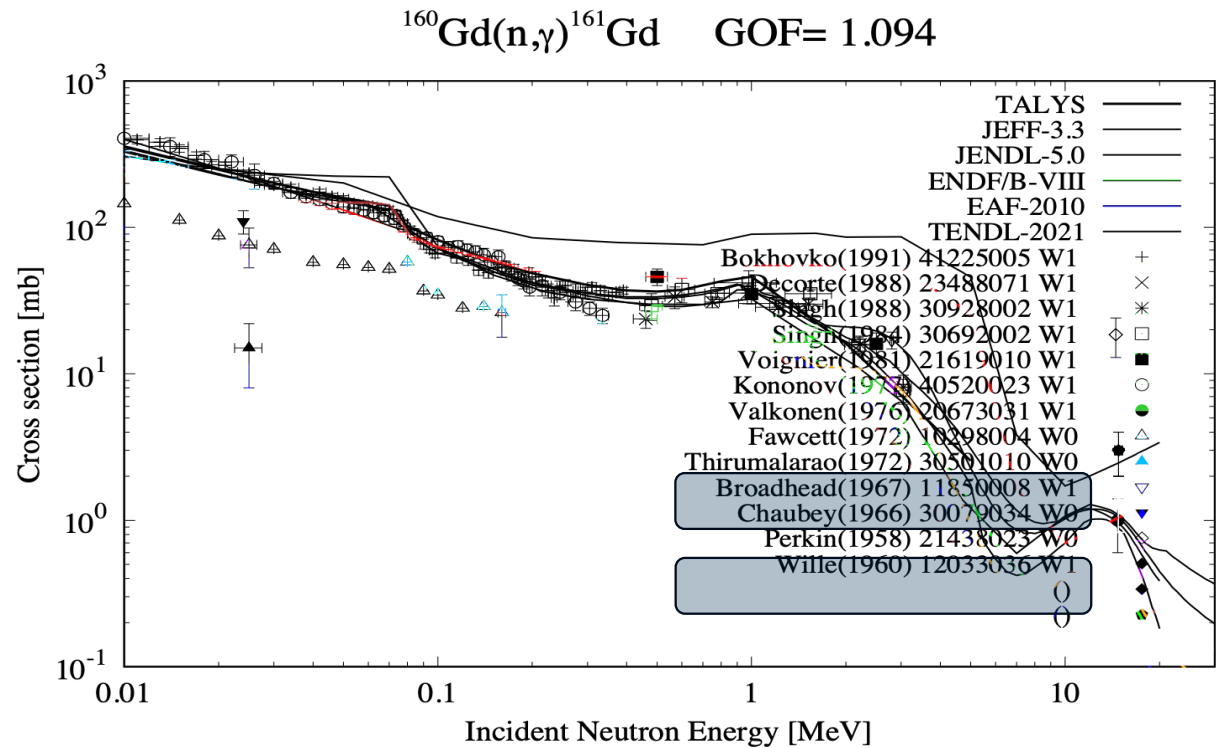
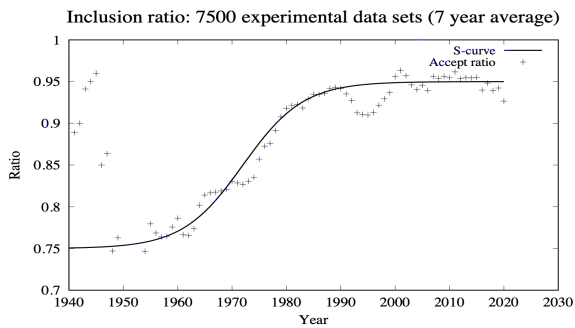
Reaction	Nuclides						
(n, γ)	278	wtable					
(n,f)	34	vfiscor	betafiscor	ctable(1)	ptable(1)	ctable(2)	ptable(2)
(n,n';2n;p)	210	rv(p)	g _{ph} (0)	g _{ph} (n)	ctable(n)	ctable(p)	
(n, α)	157	rv(α)	Cstrip(α)	g _{ph} (0)	ctable(α)		
(p,n)	142	rv(p)	rwd(p)	rv(n)	g _{ph} (0)	g _{ph} (n)	ctable(n)
(γ ,n)	77	wtable	ftable	etable			
(α ,n)	93	rv(α)	rwd(α)	rv(n)	g _{ph} (0)	ctable(α)	
(d,n)	40	rv(p)	rwd(p)	rv(n)	g _{ph} (0)	g _{ph} (n)	ctable(n)



TALYS coded input parameters & GOF

Outlier assignment (W0) and optimised TALYS model parameters per nuclide and reaction channel

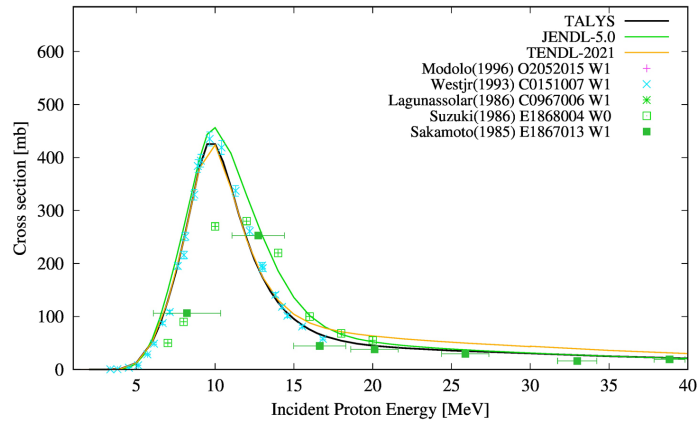
7500 experiments, age weighted, subjective, but..



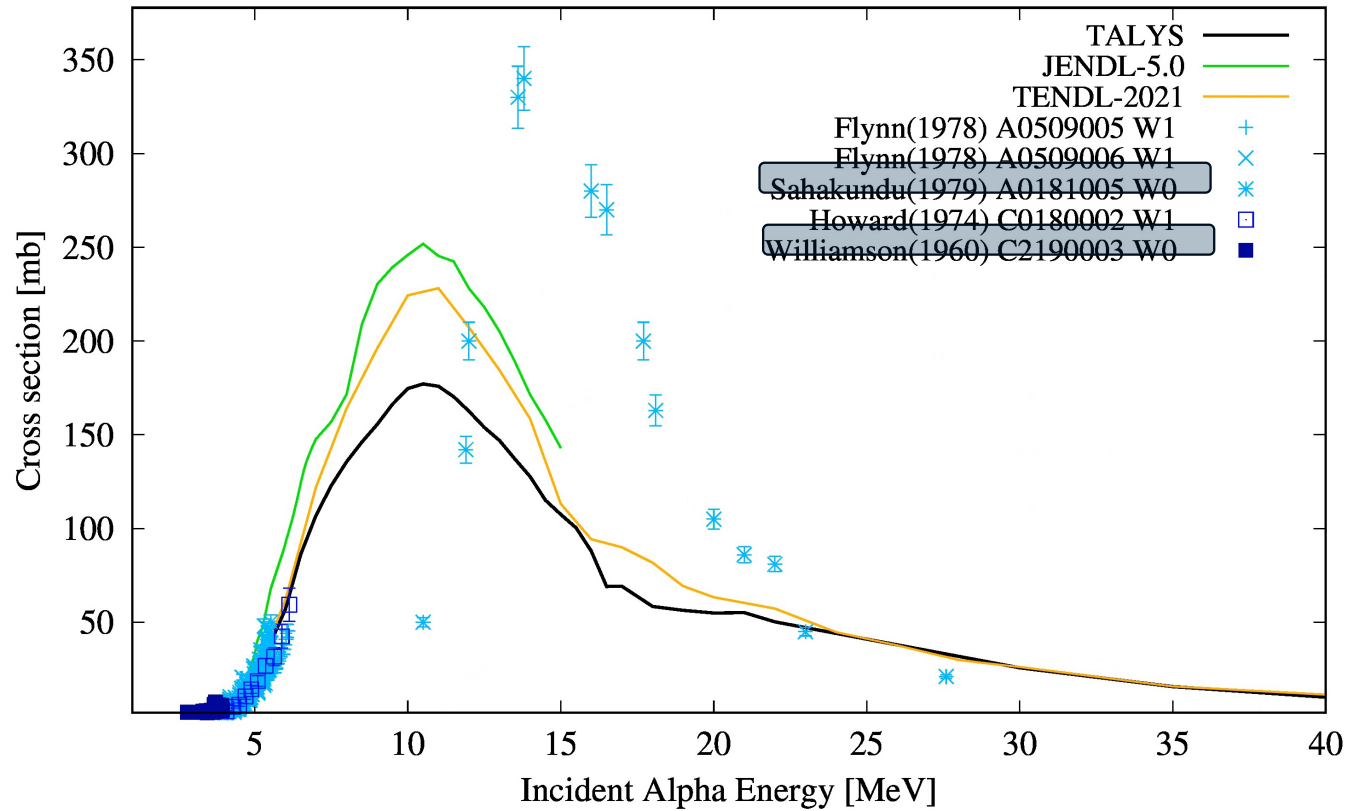
Scouting the nuclear landscapes

(α, n) , (n, α) and (p, n) films

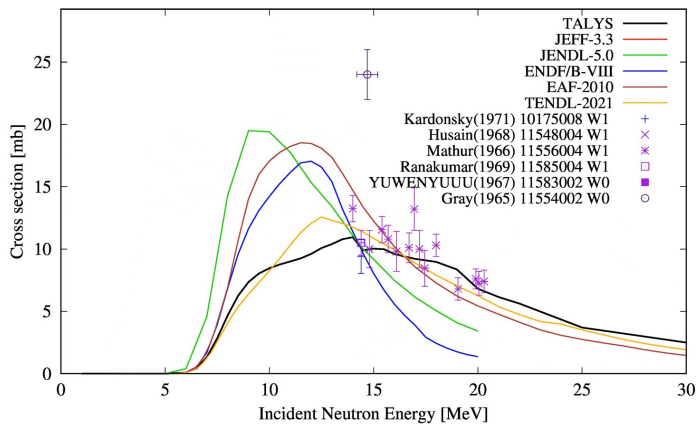
$^{127}\text{I}(p, n)^{127}\text{Xe}$ GOF= 1.33



$^{27}\text{Al}(\alpha, n)^{30}\text{P}$ GOF= 1.31



$^{40}\text{Ar}(n, \alpha)^{37}\text{S}$ GOF= 1.05



Form & Format

EXFOR



```
<reaction label="U236 + photon" ENDF_MT="102">
<crossSection>
<resonancesWithBackground label="eval">
<resonances href="/reactionSuite/resonances"/>
<background>
<resolvedRegion>
<XYS1d>
<axes>
<axis index="1" label="energy_in" unit="eV"/>
<axis index="0" label="crossSection" unit="b"/></axes>
<values>
1.0000000e-05 0.0000000e+00 2.2500000e+03 0.0000000e+00</values></XYS1d></resolvedRegion>
<fastRegion>
<XYS1d>
<axes>
<axis index="1" label="energy_in" unit="eV"/>
<axis index="0" label="crossSection" unit="b"/></axes>
<values>
2.2500000e+03 2.3282250e+00 2.5000000e+03 2.2061900e+00 3.0000000e+03 2.0307400e+00 3.5000000e+03 1.8976900
```

GNDS

```
# n + 235U : (n,g) Total
# Q-value = 6.54552E+00
# E-threshold= 0.00000E+00
# # energies = 24
# E xs gamma xs xs/res.prod.xs
1.00000E-02 2.07936E+03 1.13814E+04 1.00000E+00
2.00000E-02 1.43254E+03 7.88163E+03 1.00000E+00
4.00000E-02 1.17702E+03 6.52455E+03 1.00000E+00
7.00000E-02 9.83417E+02 5.49190E+03 1.00000E+00
1.00000E-01 8.45338E+02 4.74815E+03 1.00000E+00
2.00000E-01 5.62366E+02 3.19982E+03 1.00000E+00
```

TALYS-1.97

TITLE New precision measurements of the 235U(n,g) cross section
AUTHOR (M.Jandel, T.A.Bredeweg, E.M.Bond, M.B.Chadwick, A.Couture, J.M.O'Donnell, M.Fowler, R.C.Haight, T.Kawano, R.Reifarh, R.S.Rundberg, J.L.Ullmann, D.J.Vieira, J.M.Wouters, J.B.Wilhelmy, C.Y.Wu, J.A.Becker)
INSTITUTE (1USALAS,1USALRL)
REFERENCE (J,PRL,109,202506,2012) Final (n,g) and alpha (J,NIM/B,261,986,2007) Prelim. (n,g) and (n,f) in figs (C,2007NICE,1,607,2008) Prelim. (n,g) and (n,f) in figs
FACILITY (LINAC,1USALAS) Lujan Neutron Scattering Center of

```
# target z : 92
# Target A : 235
# Target state:
# Projectile : n
# Reaction : (n,g)
# Final state :
# Quantity : Cross section
# Frame : L
# MF : 3
# MT : 102
# X4 Subentry : 141490072
# X4 Reaction : 92-U-235(N,G)92-U-236,,SIG
# Author : Jandel
# Year : 2012
# Data points : 66
# E(MeV) xs(mb) dxs(mb) dE(MeV)
5.00036E-04 2.61329E+03 5.78510E+02 1.15100E-06
5.07024E-04 3.55298E+03 3.12190E+02 5.83701E-06
5.18834E-04 4.99961E+03 3.47930E+02 5.97351E-06
5.30919E-04 5.00055E+03 3.10070E+02 6.11151E-06
```

EXFOR tables

ENDF

```
9.223500+4 2.330248+2 0 0 0 09228 3102
6.544430+6 6.544430+6 0 0 1 1119228 3102
111 2 9228 3102
1.000000-5 0.000000+0 2.250000+3 0.000000+0 2.250000+3 2.047808+09228 3102
2.370000+3 1.711010+0 2.650000+3 1.881410+0 3.000000+3 1.828870+09228 3102
3.350000+3 1.963650+0 3.750000+3 1.838280+0 4.000000+3 1.676190+09228 3102
4.200000+3 1.534880+0 4.700000+3 1.626720+0 5.300000+3 1.145940+09228 3102
```

YANDF-0.2

TALYS 2.0 output file: rp039090.L02 and nnang.L01

'#' for direct use in various software, e.g. Gnuplot

Without '#': YAML 2 spaces indentation per level

Parsing to JSON should be easy

Only 5 main attributes for nuclear reactions

TALYS: 2 main attributes: 'parameters' and 'observables'

EXFOR: All specific metadata after the data block

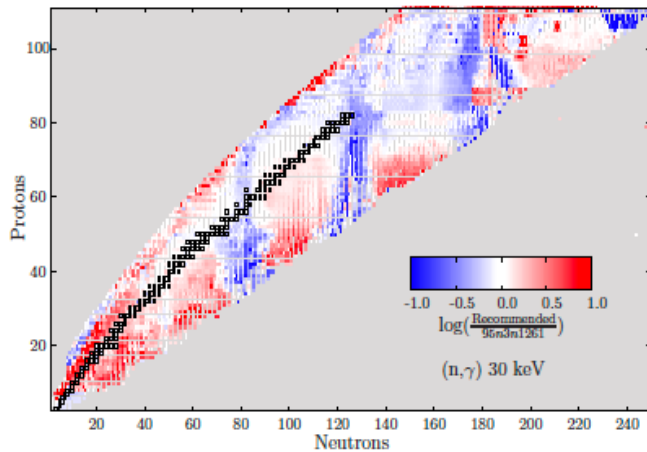
```
# header:
# title: Nb93(n,x)Y90m cross section
# source: TALYS-2.0
# user: Arjan Koning
# date: 2024-01-11
# format: YANDF-0.1
# target:
# Z: 41
# A: 93
# nuclide: Nb93
# reaction:
# type: (n,x)
# Q-value [MeV]: 4.248473E+00
# E-threshold [MeV]: 0.000000E+00
# ENDF_MF: 6
# ENDF_MT: 5
# residual:
# Z: 39
# A: 90
# nuclide: Y90m
# mass [amu]: 8.990714E+01
# level:
# number: 2
# energy [MeV]: 6.820100E-01
# spin: 7.000000E+00
# parity: 1
# isomer: 1
# half-life [sec]: 1.148000E+04
# datablock:
# quantity: cross section
# columns: 3
# entries: 25
##      E          xs      Isomeric ratio
##      [MeV]      [mb]      []
2.000000E-01  0.000000E+00  0.000000E+00
4.000000E-01  0.000000E+00  0.000000E+00
6.000000E-01  0.000000E+00  0.000000E+00
8.000000E-01  0.000000E+00  0.000000E+00
1.000000E+00  0.000000E+00  0.000000E+00
1.200000E+00  0.000000E+00  0.000000E+00
1.400000E+00  8.135896E-05  0.000000E+00
1.600000E+00  1.296808E-04  1.266436E-01
1.800000E+00  2.699952E-04  1.644318E-01
2.000000E+00  5.832345E-04  2.057970E-01
```

```
# header:
# title: C12(n,n_1) angular distribut
# source: TALYS-2.0
# user: Arjan Koning
# date: 2024-02-19
# format: YANDF-0.2
# target:
# Z: 6
# A: 12
# nuclide: C12
# reaction:
# type: (n,n_1)
# ENDF_MF: 4
# ENDF_MT: 51
# E-incident [MeV]: 5.000000E+00
# level:
# number: 1
# energy [MeV]: 4.439820E+00
# spin: 2.000000E+00
# parity: 1
# datablock:
# quantity: angular distribution
# columns: 4
# entries: 91
##      Angle          xs      Direct      Compound
##      [deg]          [mb/sr]  [mb/sr]    [mb/sr]
0.000000E+00  1.676661E+01  5.253240E-01  1.624129E+01
2.000000E+00  1.676645E+01  5.252810E-01  1.624117E+01
4.000000E+00  1.676595E+01  5.251530E-01  1.624080E+01
6.000000E+00  1.676513E+01  5.249390E-01  1.624020E+01
8.000000E+00  1.676399E+01  5.246370E-01  1.623935E+01
1.000000E+01  1.676251E+01  5.242470E-01  1.623827E+01
1.200000E+01  1.676072E+01  5.237660E-01  1.623695E+01
1.400000E+01  1.675860E+01  5.231930E-01  1.623540E+01
1.600000E+01  1.675616E+01  5.225260E-01  1.623363E+01
1.800000E+01  1.675340E+01  5.217600E-01  1.623164E+01
2.000000E+01  1.675033E+01  5.208940E-01  1.622944E+01
2.200000E+01  1.674695E+01  5.199230E-01  1.622703E+01
2.400000E+01  1.674327E+01  5.188450E-01  1.622442E+01
```

TENDL-2023 ASTRO https://tendl.web.psi.ch/tendl_2023/astro/astro.html

Started August 10th, 2022 and still goin on...~8000 nuclides

200 cores, up to 480 model combinations x isotopes x 4 metrics



model 95n3n1261

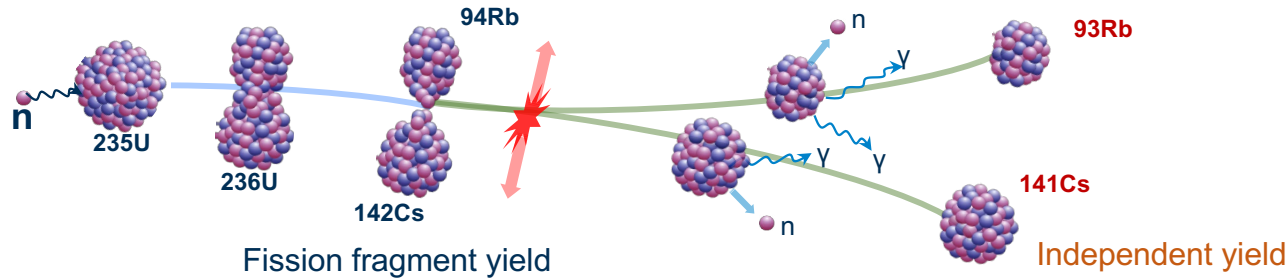
TENDL-astro 2023

Cross sections, reaction rates and MACS for astrophysics

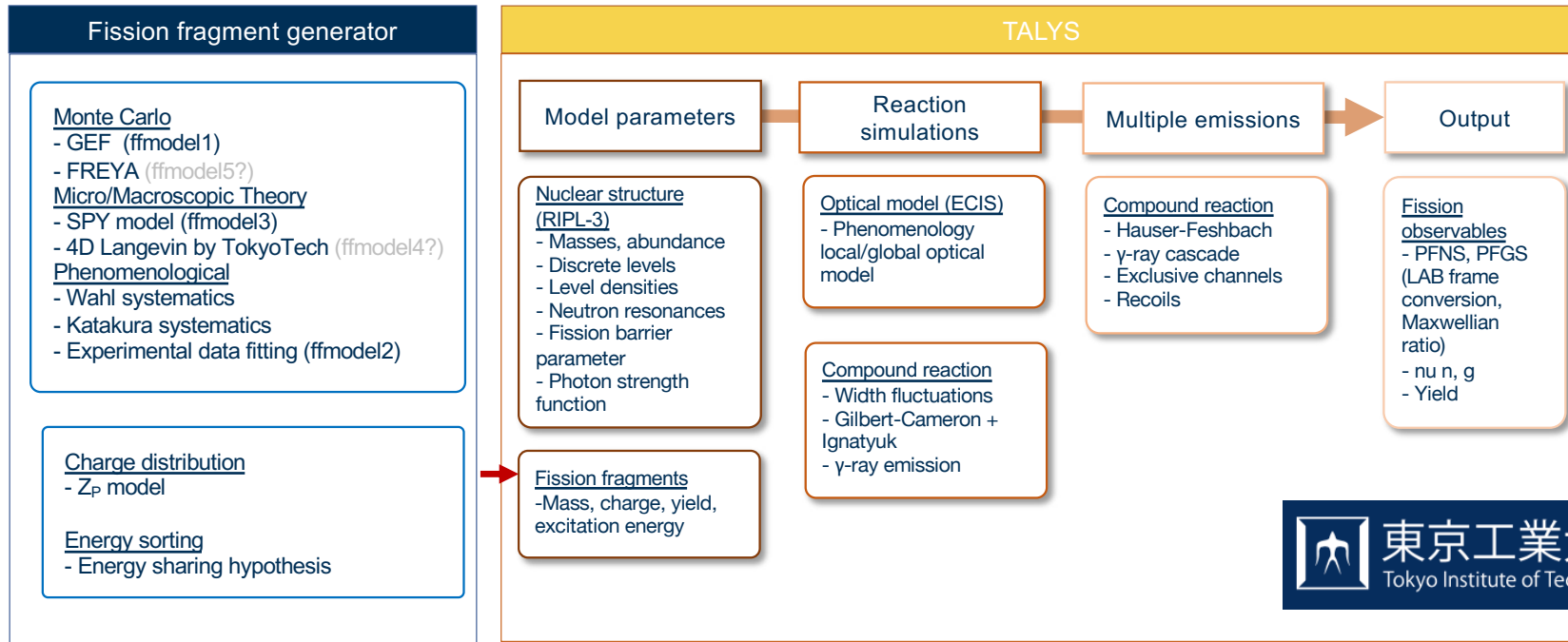
Quantities with uncertainties

	10 models	480 models (A < 210)	960 models (A ≥ 210)
1. cross sections:	(n,g) (n,p) (n,a)	(n,g) (n,p) (n,a)	(n,g) (n,p) (n,a)
2. cross sections:	(p,g) (p,n) (p,a)	(p,g) (p,n) (p,a)	(p,g) (p,n) (p,a)
3. cross sections:	(a,g) (a,n) (a,p)	(a,g) (a,n) (a,p)	(a,g) (a,n) (a,p)
4. reaction rates:	(n,g) (n,p) (n,a)	(n,g) (n,p) (n,a)	(n,g) (n,p) (n,a)
5. reaction rates:	(p,g) (p,n) (p,a)	(p,g) (p,n) (p,a)	(p,g) (p,n) (p,a)
6. reaction rates:	(a,g) (a,n) (a,p)	(a,g) (a,n) (a,p)	(a,g) (a,n) (a,p)
7. Normalization function:	G(T)	G(T)	G(T)
8. Maxwellian Averaged (n,g) :	MACS	MACS	MACS

TALYS: prompt fission observable and fission yields



TALYS treats a fission fragment as an excited nucleus and applies the Hauser-Feshbach statistical decay to the fission fragment de-excitation (prompt decay) and loops over all fission fragments.

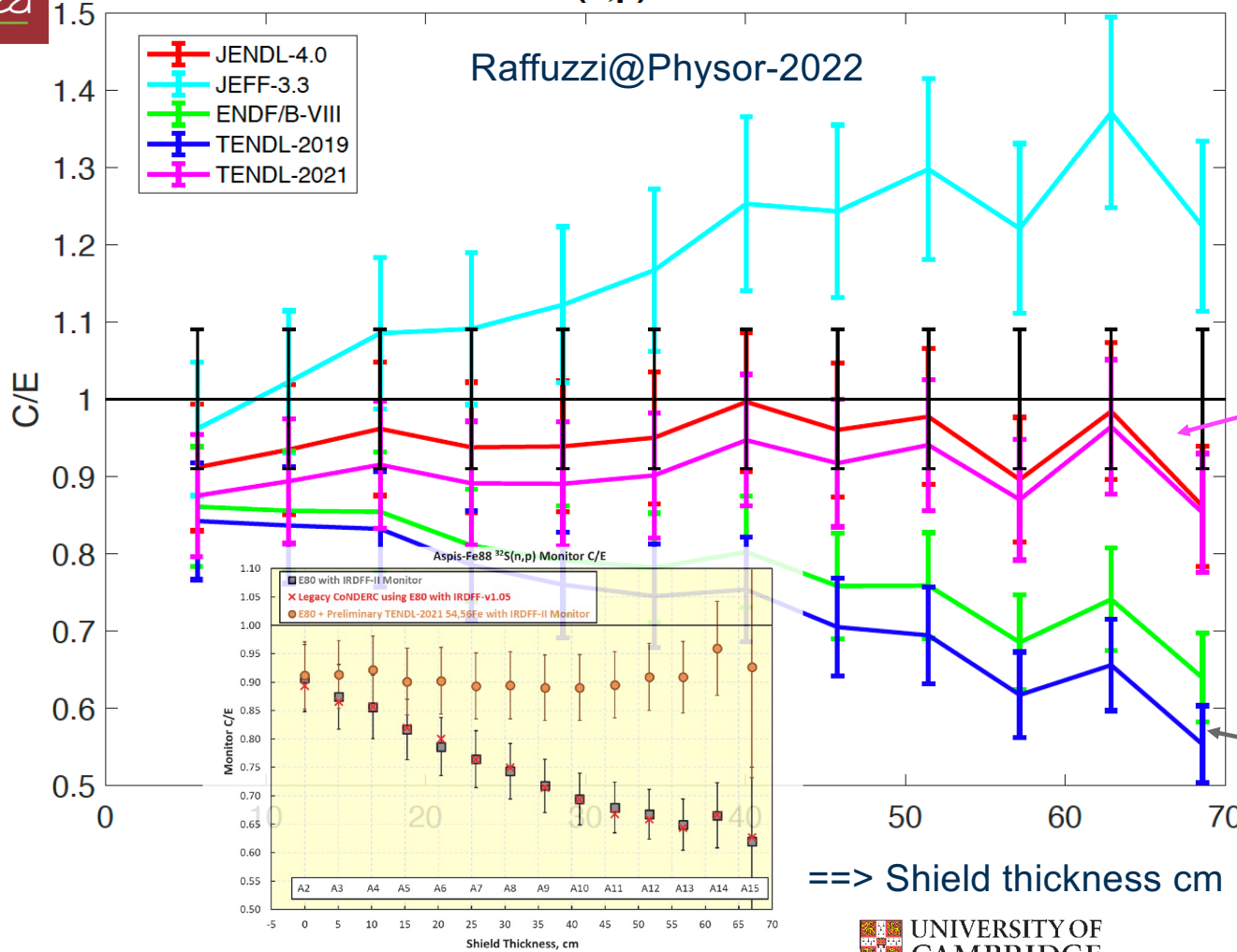


V&V TRIPOLI4® & MCNP6® Shielding ASPIS

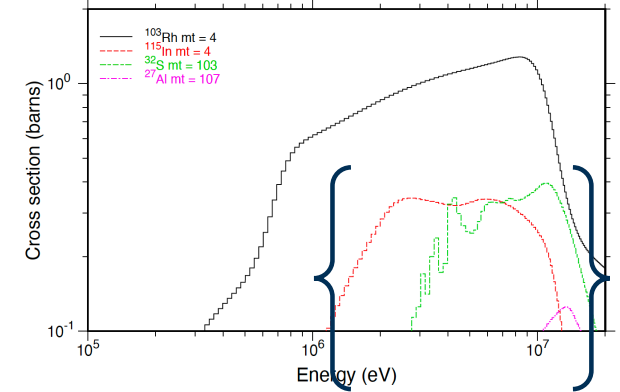


$^{32}\text{S}(n,p)$ - First file

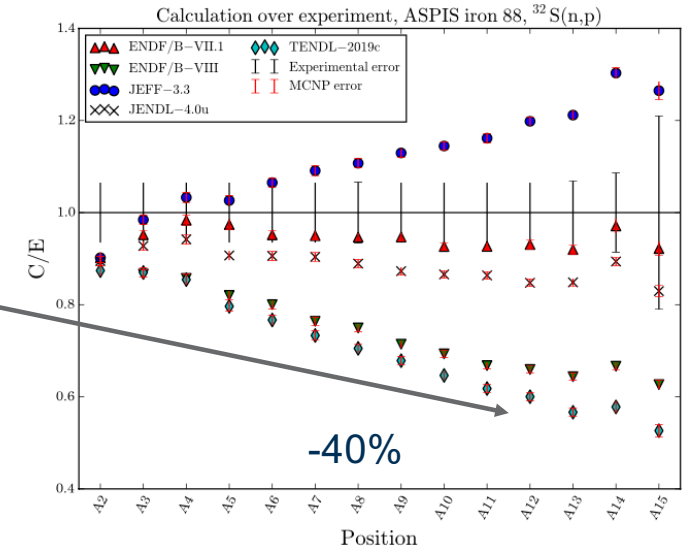
Raffuzzi@Physor-2022



IRDFF-II Flux Monitors



TENDL-23 as performant as JENDL-4

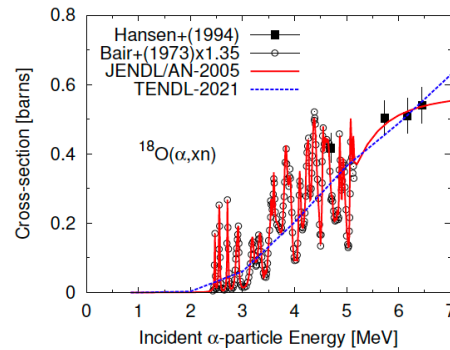
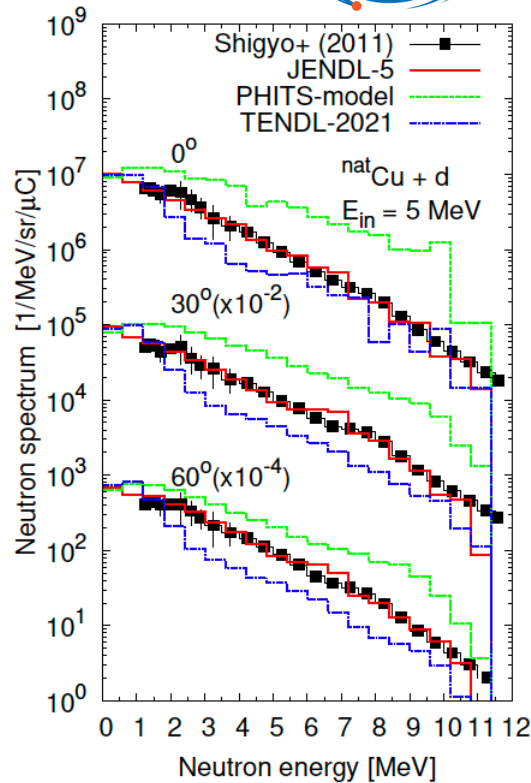


==> Shield thickness cm

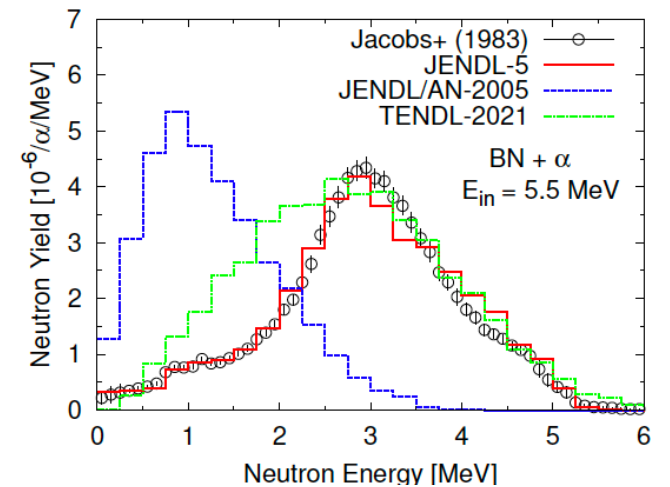
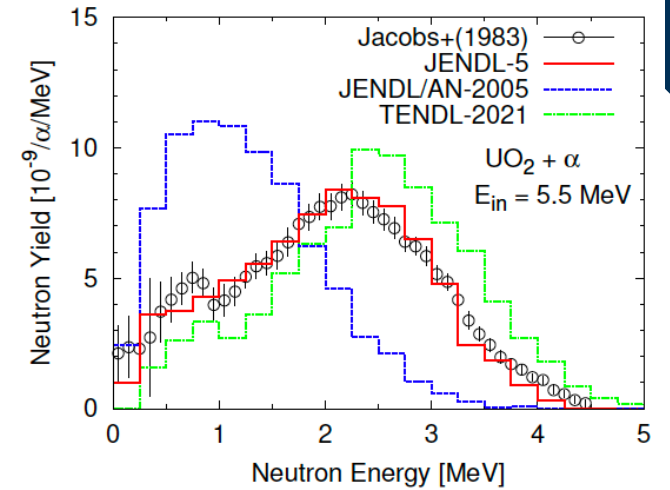


CP's V&V deuteron, alpha, gamma induced libraries

5 MeV deuteron on thick Copper

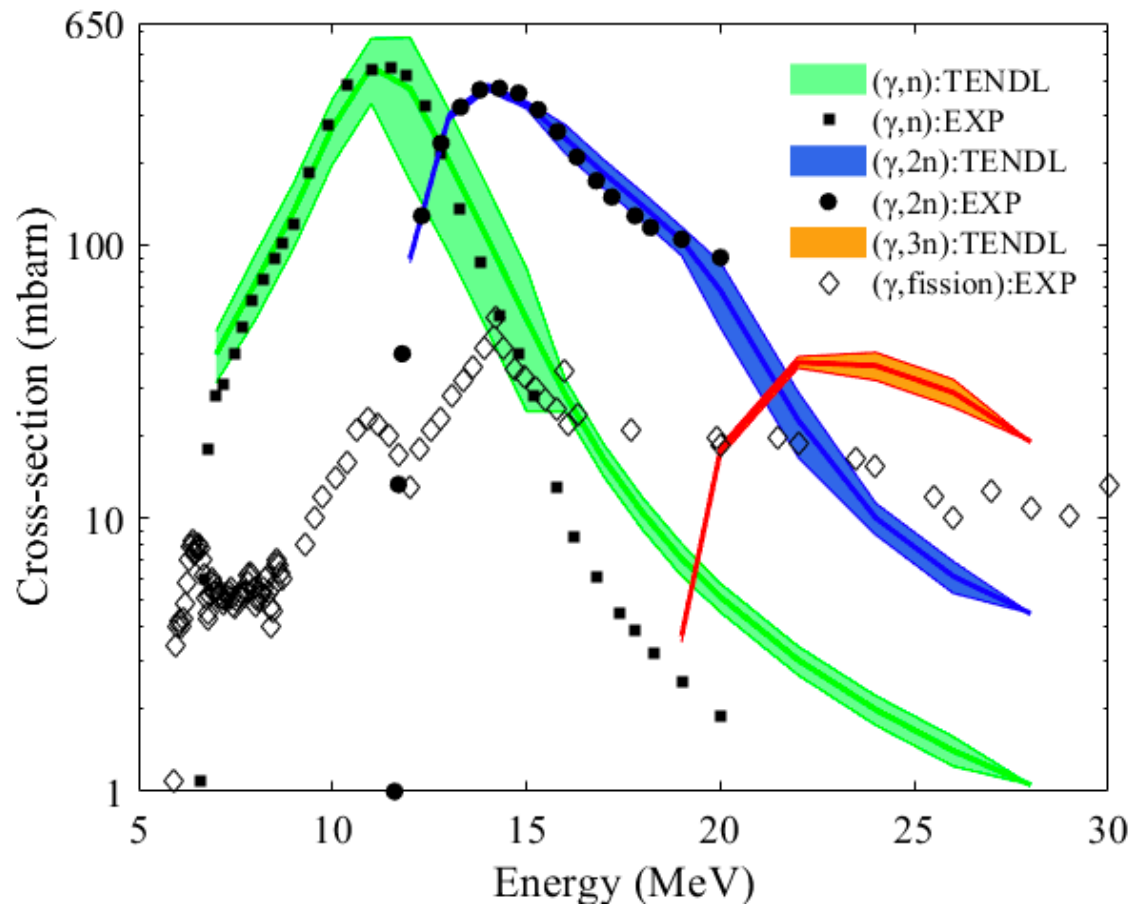


- 5.5 MeV alpha on UO₂ and Boron nitride



CP's V&V deuteron, alpha, gamma induced libraries

- Gamma on ^{232}Th



Caveat:
 γ -fission in MF-10 !!

TENDL-2023 innovation

Usage of an enhanced, complete, resonance parameter database

Major update of the experimental differential and integral databases

Covariances information on all **seven incident particles**

- from 2 to 100 TASMEN (depending on the half-life of the target nuclei)

Re-written, modernized codes, GitHub accessible

Autotalys default parameters script enhancement, verification

```
$autotalys -element Na -mass 24 -proj n -E30 -nocovar >& n-Na024.log
```

```
$loopnuc
```

```
$allnuccheck
```

200 cores times ~3 months, Swiss power @



ENDF, GNDS forms processes

Processing system convert ENDF, GNDS forms into forms useful for **practical applications**: fission, fusion, stockpile stewardship, criticality safety, radiation shielding, nuclear medicine procedures, earth, space exploration and more

Processed nuclear data forms are numerous, rich, abundant, diverse. Some are observable, other not, all have a specific importance for **at least one application: GIDI+: GNDS to GEANT4 NDL, and LLNL's ADRDA, MERCURY MC**

Processing **enhances, enriches, deepens** the evaluated nuclear data to **forms useful for applications** and **well beyond** cross-section or pile criticality studies only

ENDF, GNDS tabulated forms

Evaluated forms are **hybrid specimen** assembled from

experimental information & R Matrix formalism

nuclear model & structure

nuclear model & structure & differential tweak

nuclear model & structure & differential tweak & integral adjustment

Evaluated forms are just a commencement, although it is seen as an end by an entire community

The laws of Physics allow many verification (not validation) processes to take place during the making, assemblage and processing of evaluated file: T3 infrastructure