

Investigating Thermal Neutron Radiation Shielding Features of B2O3 and Gd2O3-doped Material (Quartz, Glasses, Al, W): A Monte Carlo Simulation



M.Kamislioglu* (1), E.Tiras (2,3), A.Bat (2)

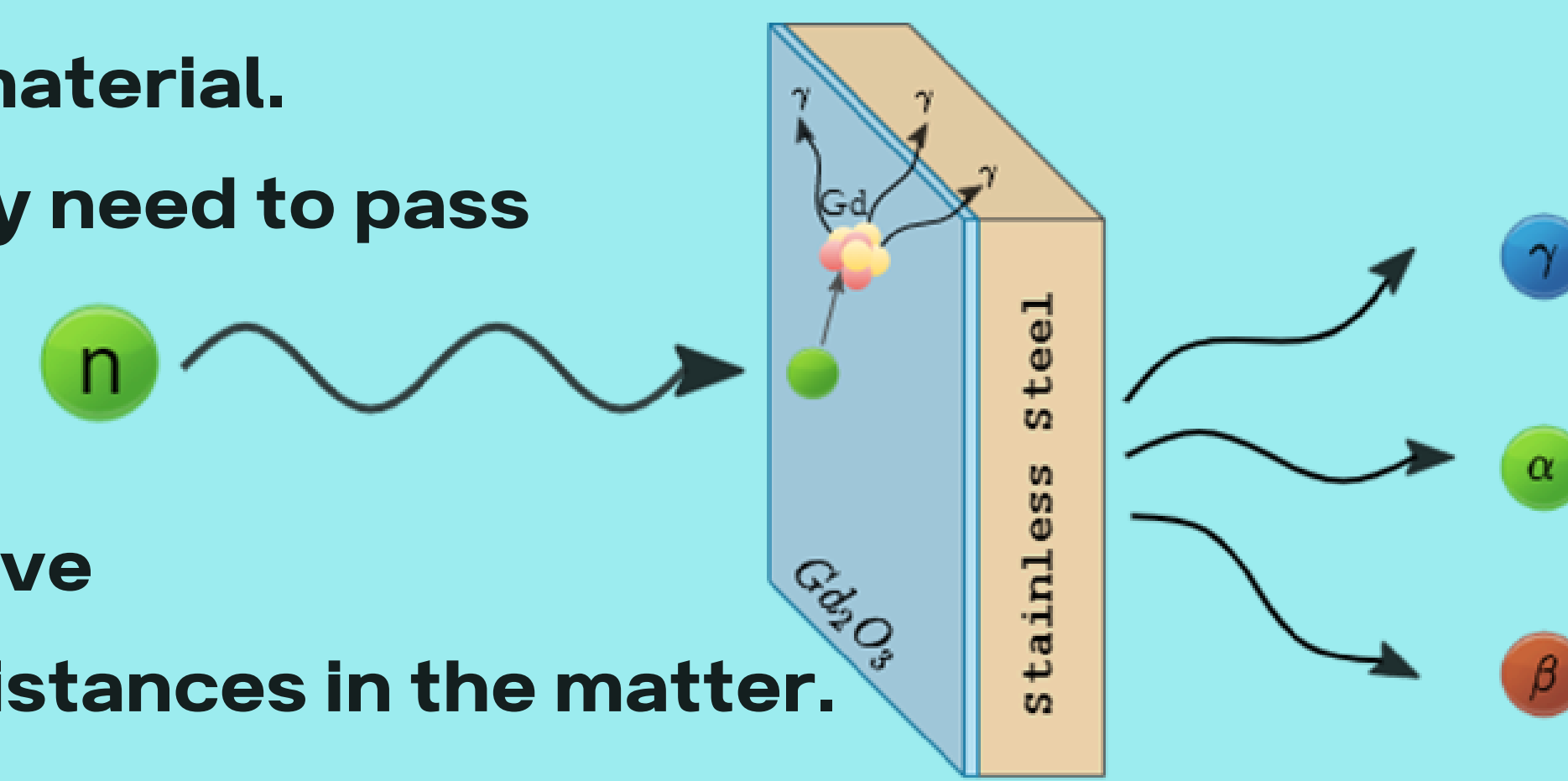
(1) Bandirma Onyedi Eylul University (2) Erciyes University, (3) The University of Iowa



Introduction

Neutrons have no net electric charge so they cannot be affected or stopped by electric forces.

- Neutron interaction energies from 1 eV to 100 MeV.
- No interaction with the atomic electrons of the material.
- They interact with the nuclei of the atoms so they need to pass closer to the nuclei.
- The nucleus radius is about 10^{-13} cm.
- Due to the smallness of the nucleus, neutrons have low probability of interaction and long traveling distances in the matter.



Results

- Like gamma radiation, neutrons undergo scattering and absorption interactions with matter.
- These interactions form the basis for methods used to shield and measure neutron radiation.
- Neutron interactions are obtained by considering all interaction mechanisms.
- The results obtained by using the equations below are shown in the Tables for the materials.

$$\sum_R = \sum_i \rho_i (\sum R/\rho)_i$$

$$\sum_R / \rho = \sum_i W_i (\sum R/\rho)_i$$

Material	$\sum R$ (cm ⁻¹)
Al	0,07911
W	0,0297
Stainless Steel	0,154039
PEN	0,095786
PET	0,096538
Quartz	0,082512

Glasses	$\sum R$ (cm ⁻¹)
C0	0,120686
C1	0,118975
C2	0,117855
C3	0,11684

Motivation

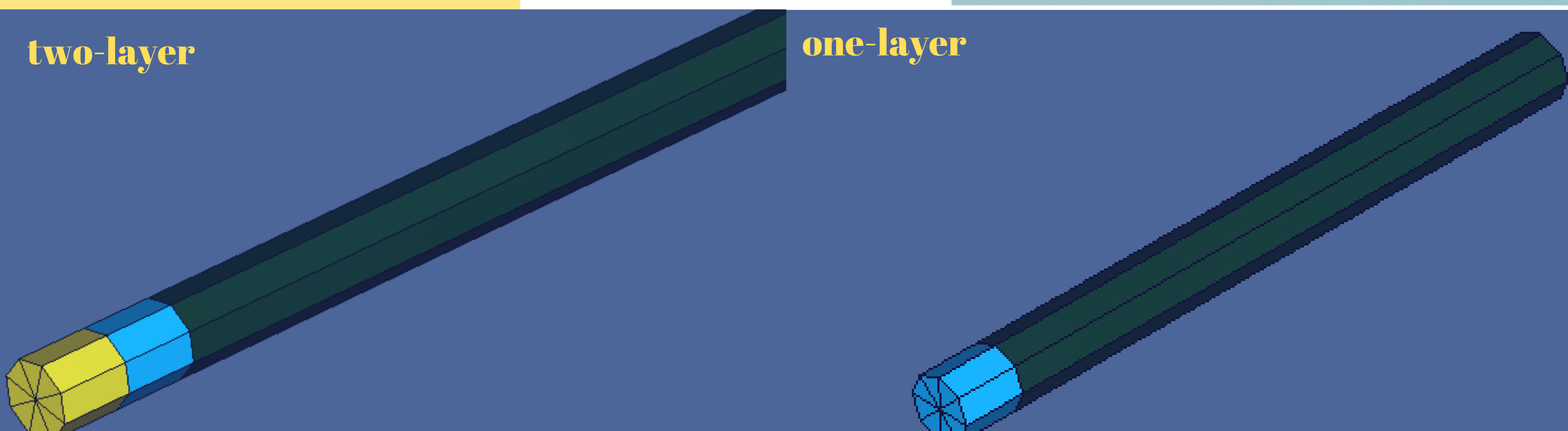
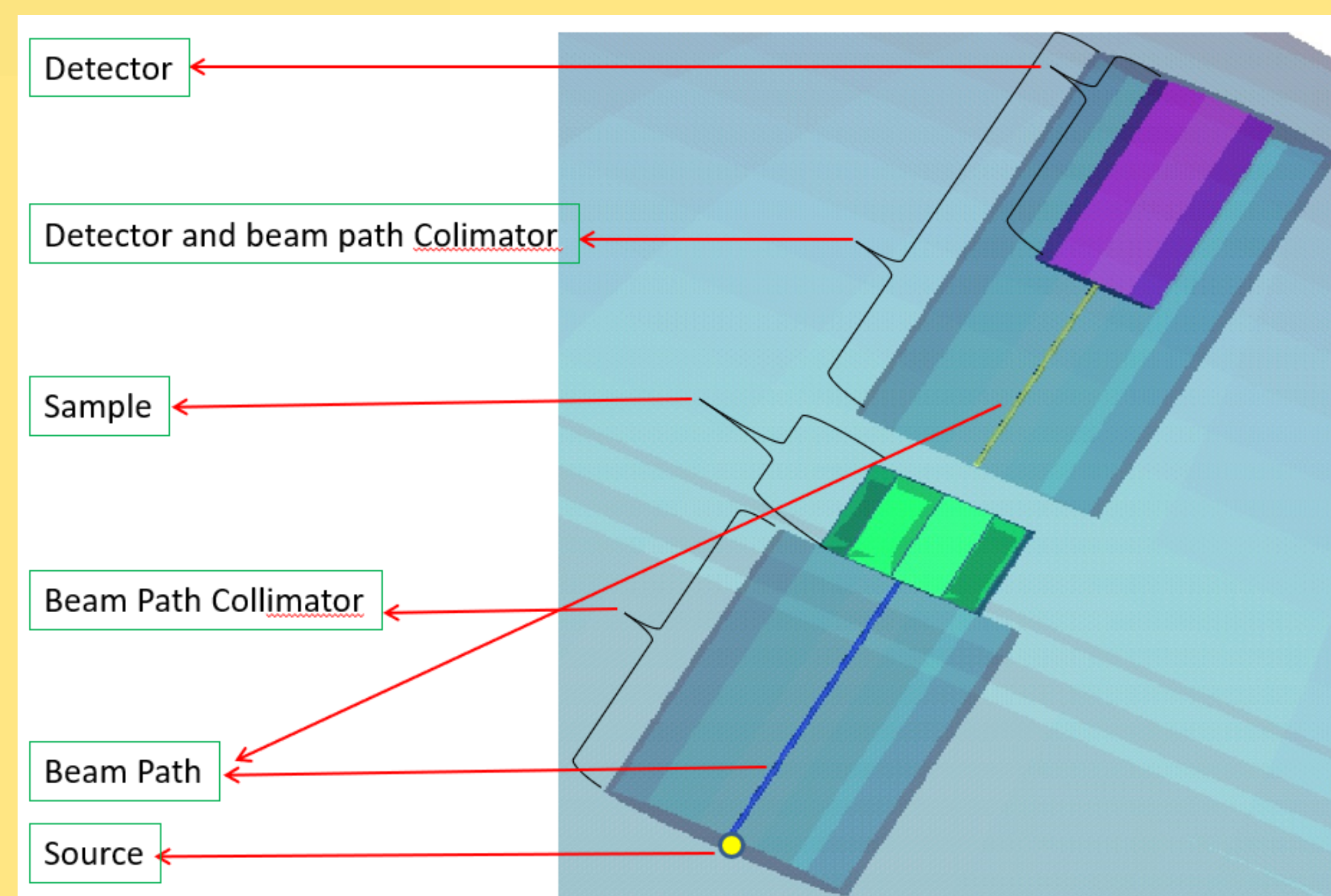
The motivation for this study is twofold:

- Studying the behavior of various materials against gamma and neutron radiation and the calculation of the effect of distance.
- Monitoring the behavior of neutrons for which the interaction mechanism has not yet been established on a mathematical basis.

MC Simulation Method

- In this study, MCNP6.2 was used for generating and transporting radiation particles through material geometry.
- MCNP can handle interactions and estimate output quantities.
- MCNP was developed in Los Alamos National Laboratory for general-purpose radiation interactions and offers the ability to transport various types of particles in 3D geometries.

The experimental setup created using MCNP6.2

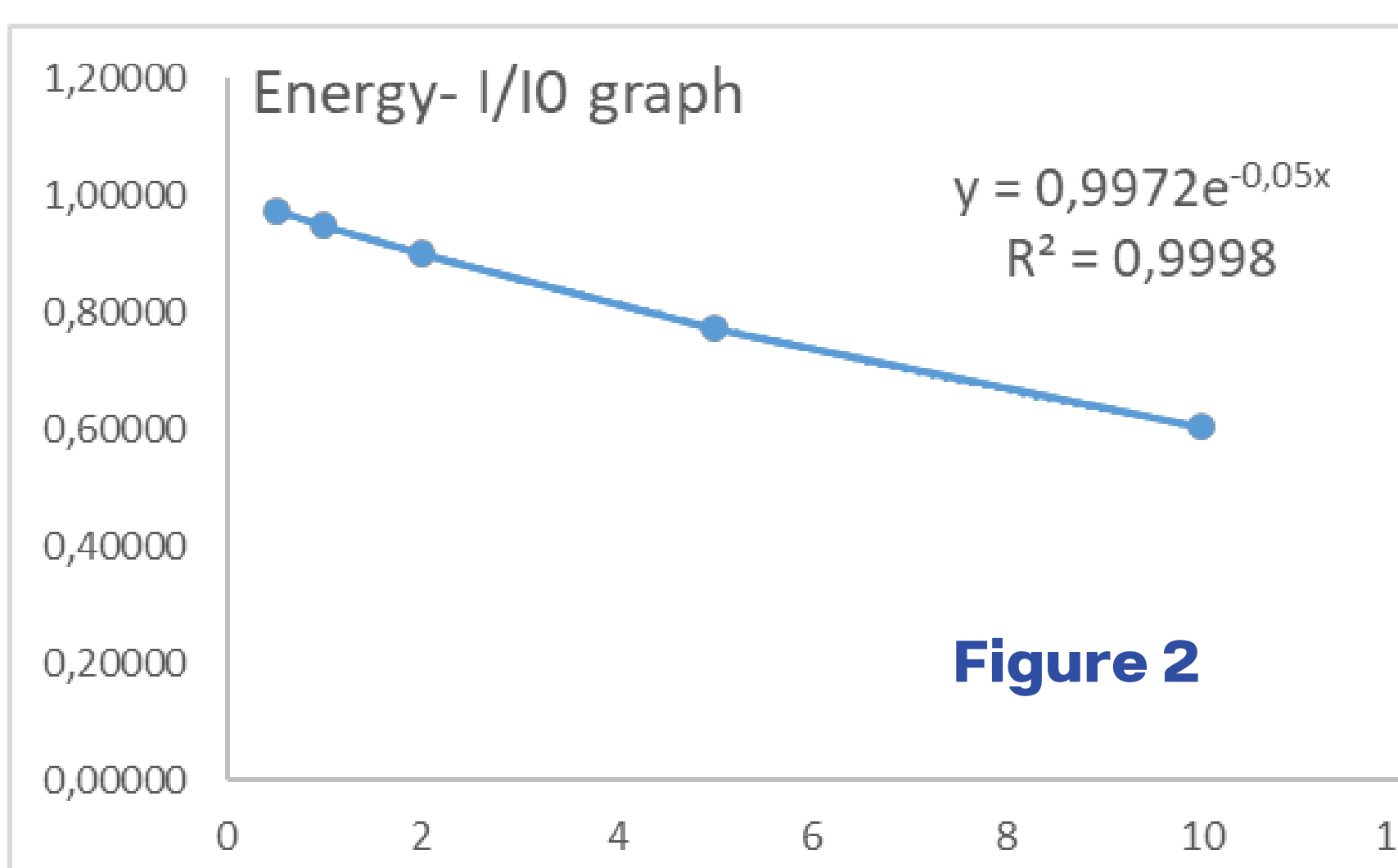
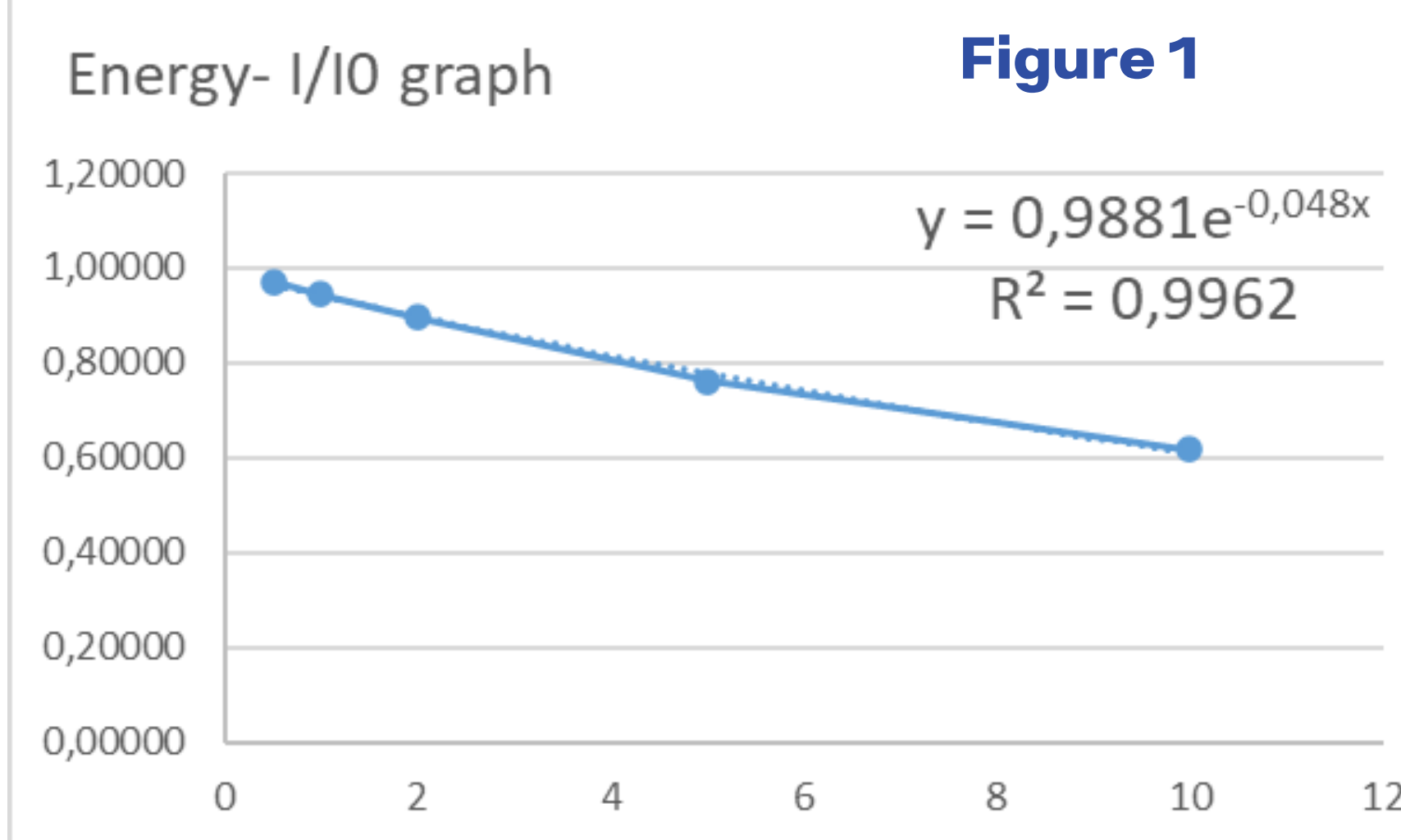


B2O3 and Gd2O3

doped Material both one-layer and two-layers.

Conclusion

- The tungsten (W), which has a high atomic number shows the best performance in the study.
- Among the glasses, C0 glass was found to have the highest neutron radiation protection feature.
- The simulation studies and analysis is still ongoing for more materials and mixtures to choose the best materials for calorimetric studies.



The removal cross-sections $\sum R$ for fast neutrons for the investigated materials and glasses are in Figure 3 and 4.

The energy- I/I0 values obtained as a result of the simulation are given in Fig. 1 and 2.

- It can be seen that Gd2O3 and B2O3-doped materials have an important role in nuclear studies.
- Total atomic cross-sections $\sum R$ offer a great opportunity to compute the energy range of heavy particles in B/Gd-doped glasses.

