





Measurement of the ²³⁵U(n,f) cross section at n_TOF from thermal to 170 keV

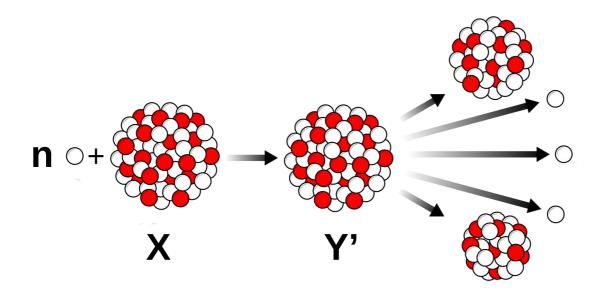








Nuclear fission



Fragments + neutrons

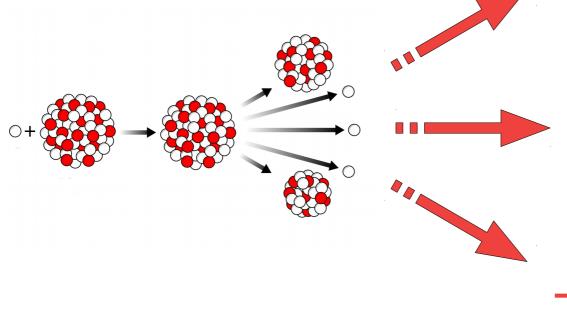




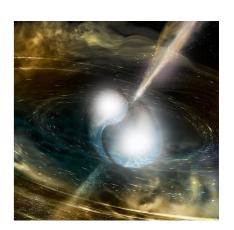
Nuclear fission



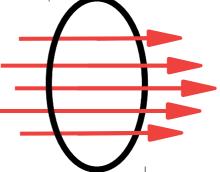
New generation fission reactors and waste burning



Heavy elements nucleosynthesis



Neutron flux measurement







Neutron standard cross sections

Well know neutron cross section used to measure flux and as reference in other neutron cross section measurement.

Reaction	Energy interval
H(n,n)	1 keV - 20 MeV
³He(n,p)	thermal - 50 keV
⁶ Li(n,t)	thermal - 1 MeV
¹⁰ B(n,α)	thermal - 1 MeV
¹⁰ B(n,α ₁ γ)	thermal - 1 MeV
C(n,n)	up to 1.8 MeV
¹⁹⁷ Au(n,γ)	thermal and 0.2 MeV - 2.5 MeV
²³⁵ U(n,f)	thermal and 0.15 MeV - 200 MeV
²³⁸ U(n,f)	20 MeV - 200 MeV



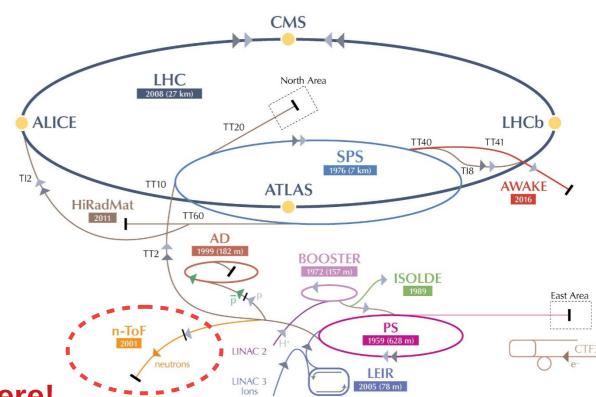




n_TOF

neutron_TimeOfFlight

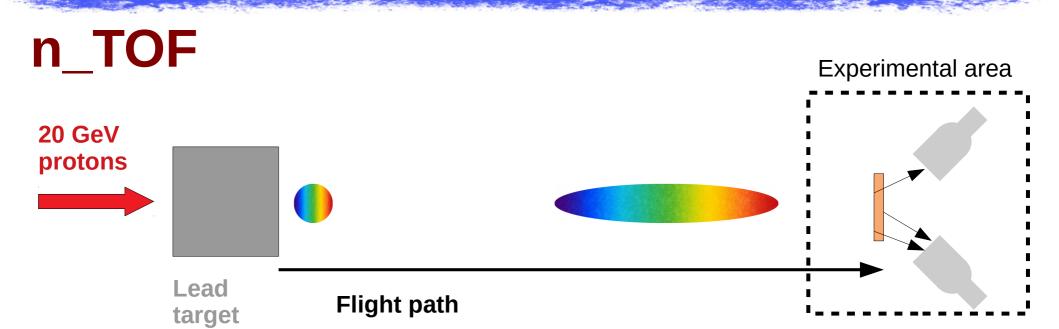




We're here!

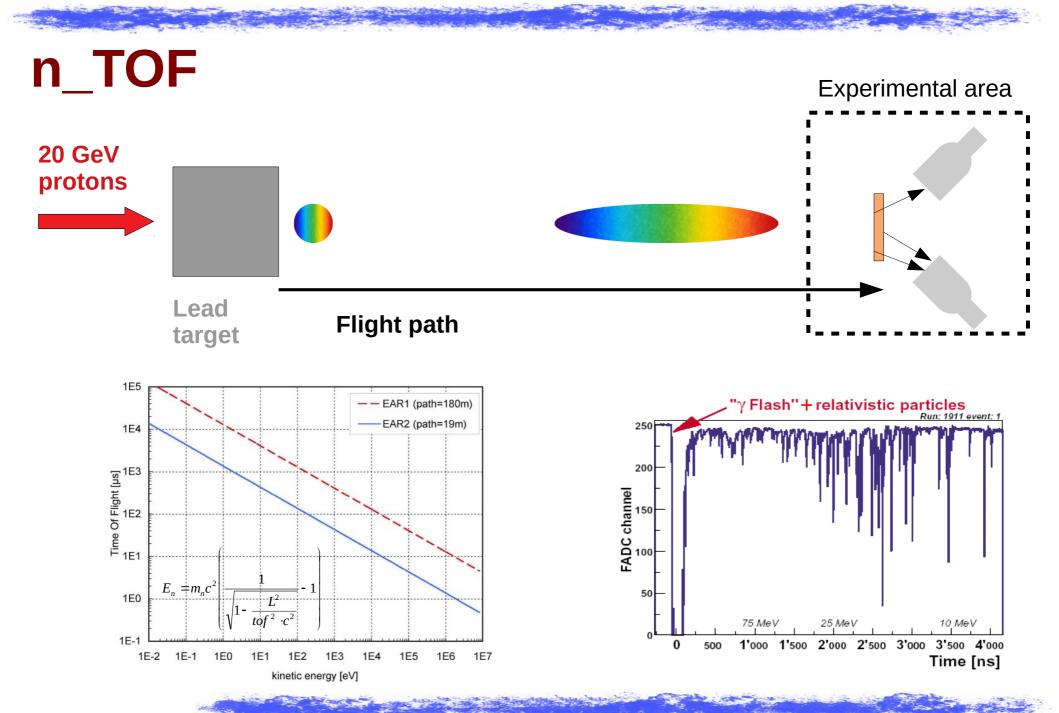










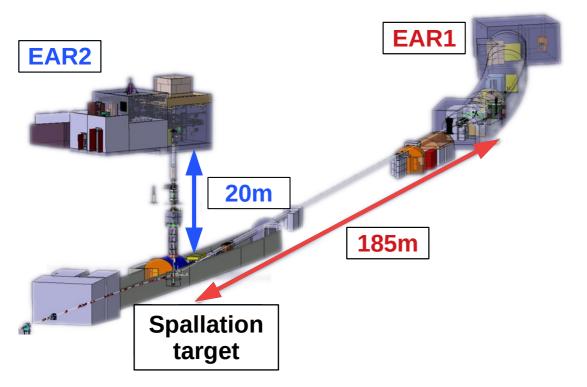


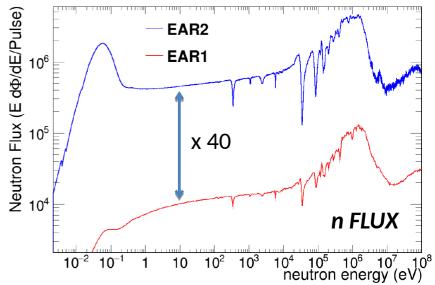






n_TOF



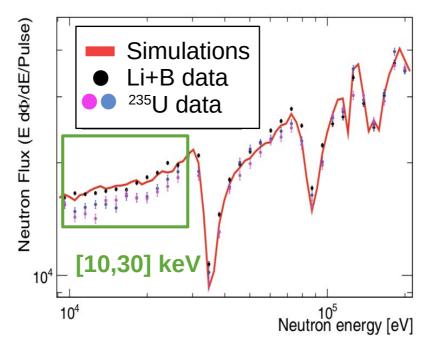






n_TOF flux measurement

Discrepancies in the keV region were found during the last n_TOF flux measurement in EAR1.



M. Barbagallo et al., Eur. Phys. J. A 49 (2013) 156



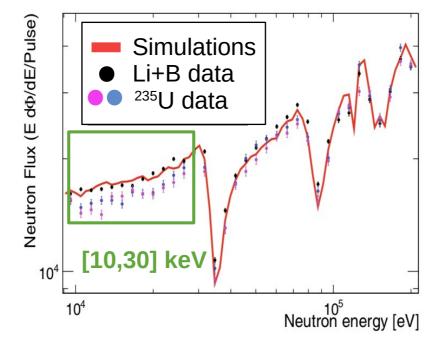


n_TOF flux measurement

Discrepancies in the keV region were found during the last n_TOF flux measurement in EAR1.

That's suspicious...





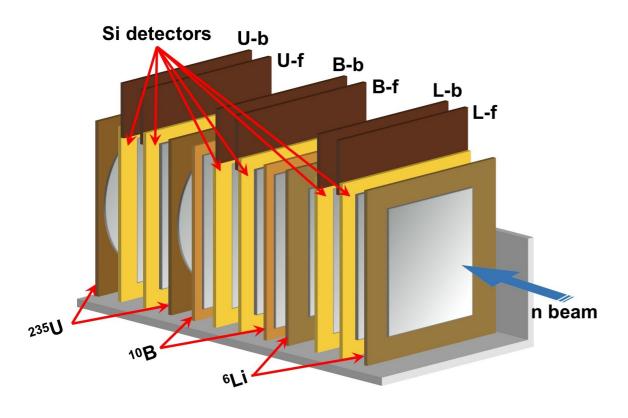
M. Barbagallo et al., Eur. Phys. J. A 49 (2013) 156

²³⁵U(n,f) cross-section measurement performed at n_TOF with respect to standards 6 Li(n,t) and 10 B(n, α) in the energy range th-170 keV.



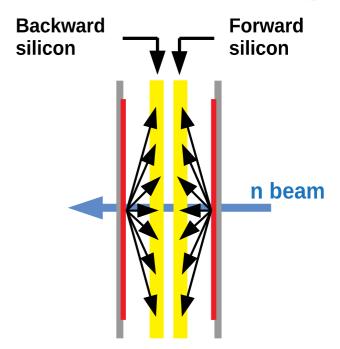


Experimental setup



Silicon thickness: 200 um Silicon area: 5x5 cm²

Distance silicon-target: ~1 mm



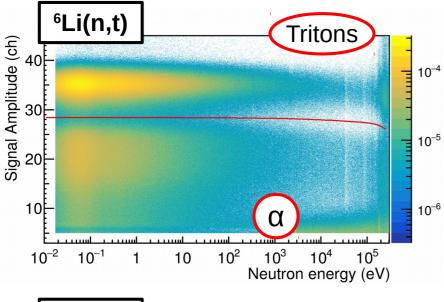


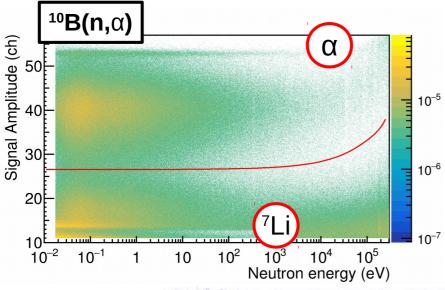






Particle identification



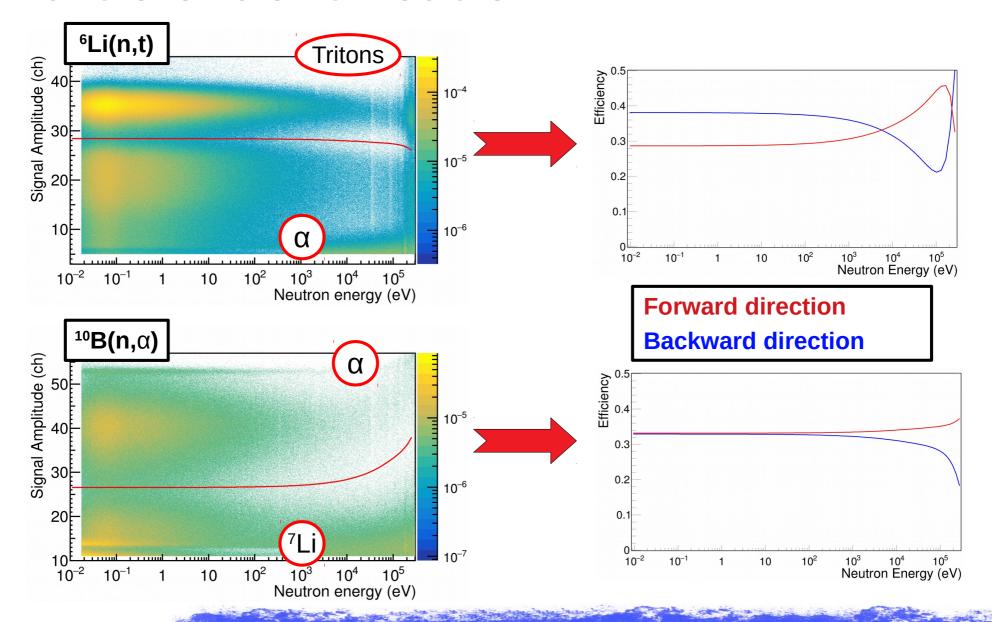








Particle identification

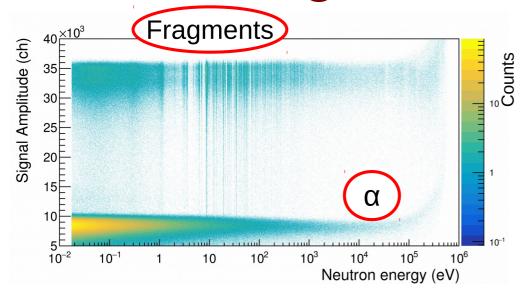








Fission Fragments discrimination



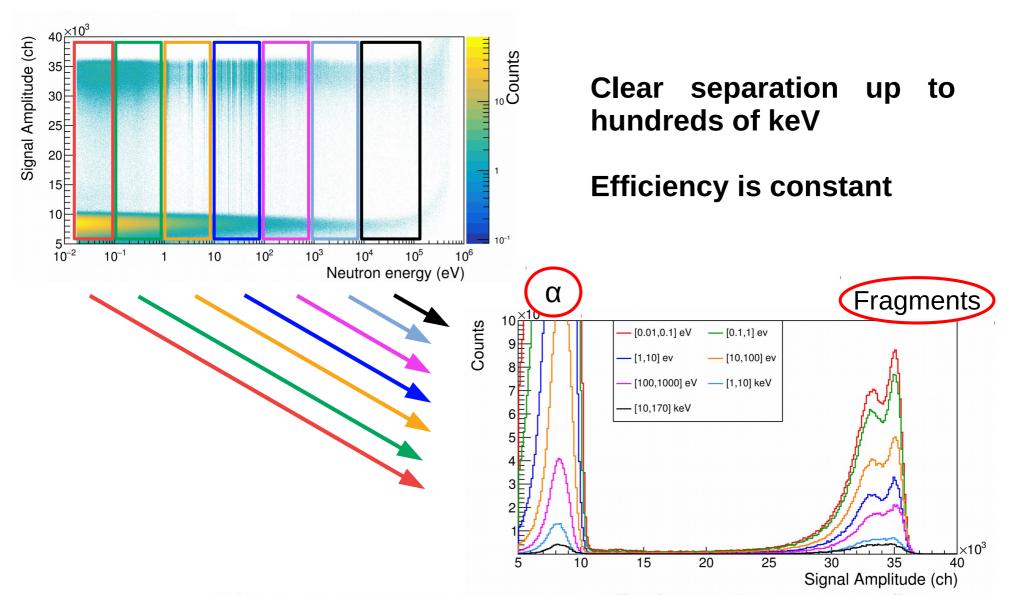
Clear separation up to hundreds of keV

Efficiency is constant





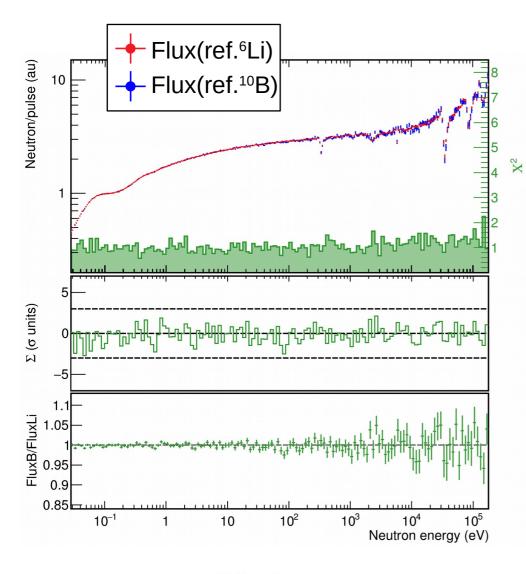
Fission Fragments discrimination



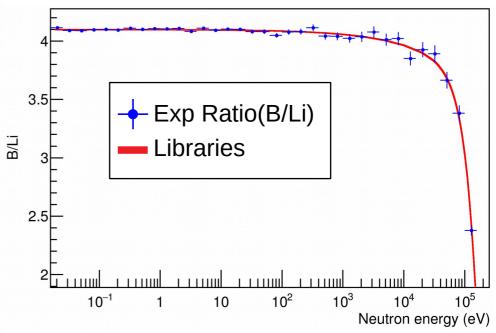




Measured flux – Ratio B/Li



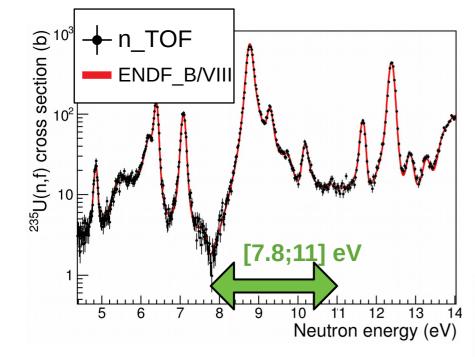
Good agreement between boron and lithium data.







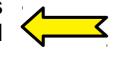
Normalization to [7.8,11]eV

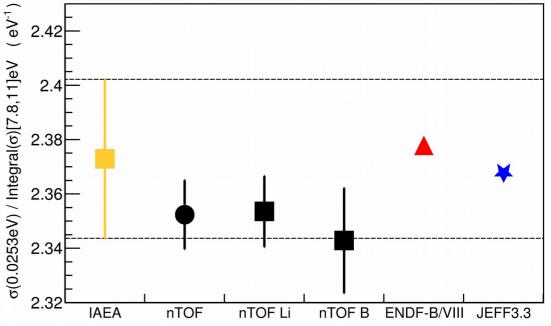


Negligible border effect (two minimum)

Integral [7.8,11]eV is a standard

Integral [7.8,11] eV is consistent with thermal value





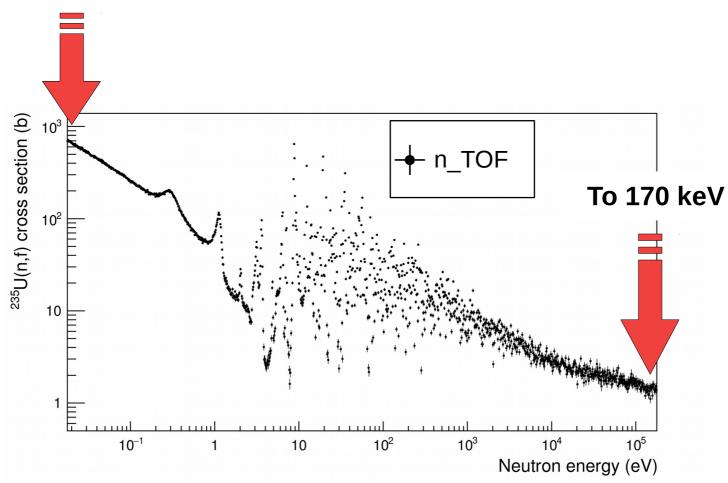






²³⁵U(n,f) cross section

From thermal energy

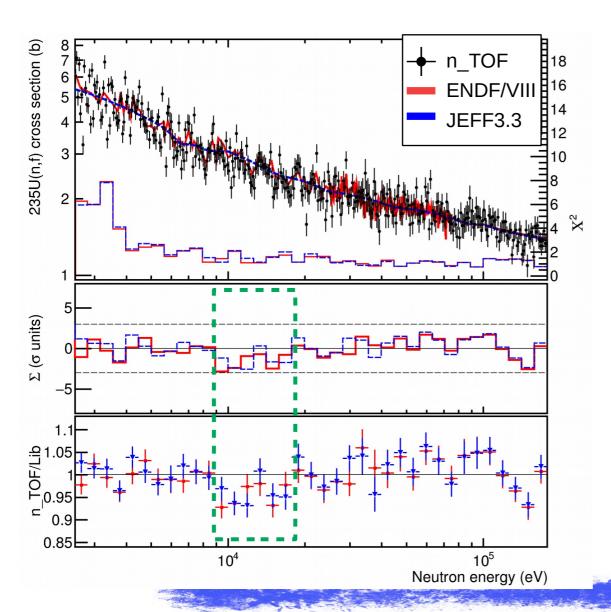








²³⁵U(n,f) in the keV Region



[9,18] keV

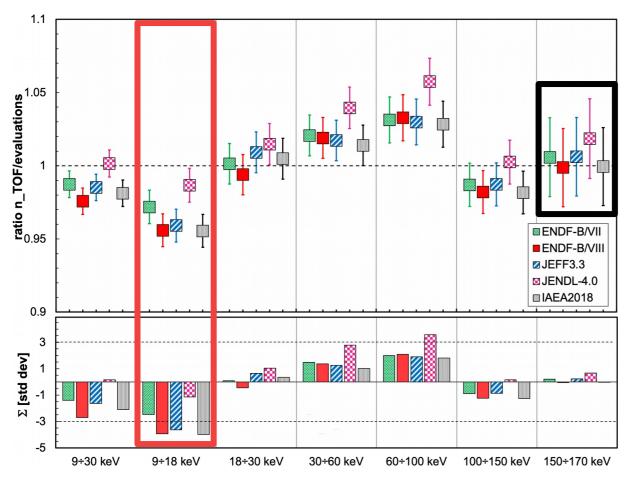
Experimental data are systematically lower than JEFF3.3 and ENDF-B/VIII







²³⁵U(n,f) in the keV Region



Over 150 keV ²³⁵U(n,f) is again a standard.

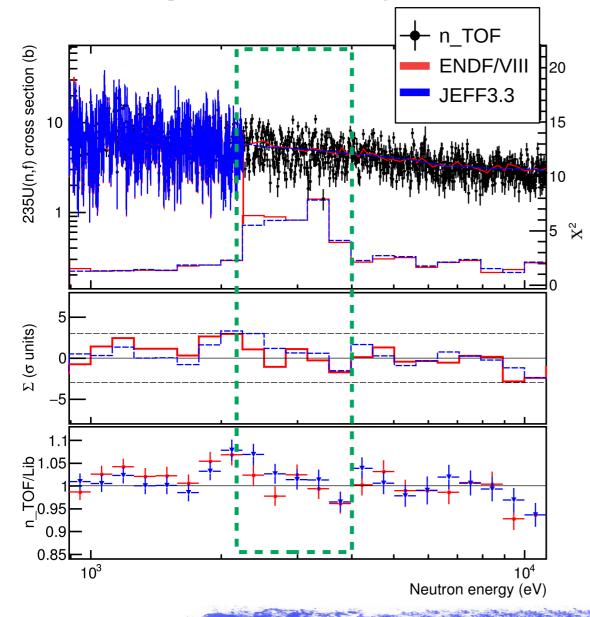
Most of recent the libraries overestimate ²³⁵U(n,f) cross section between 9 and 18 keV.







RRR-URR limit



[2.2,4] keV

Large X² between experimental data and libraries.

No systematic deviation.

Ratio exp/libraries ~ 1



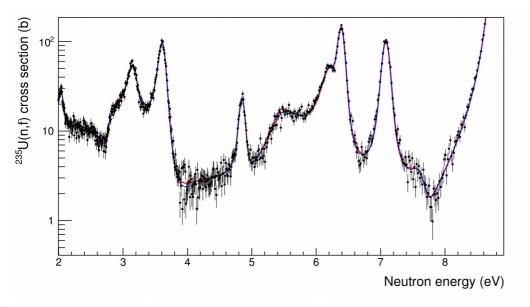
Evidences of structures in experimental data that are not present in libraries.



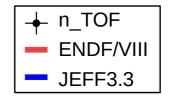


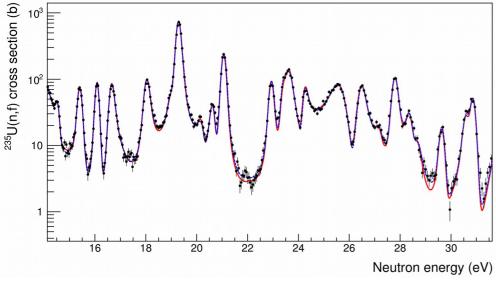


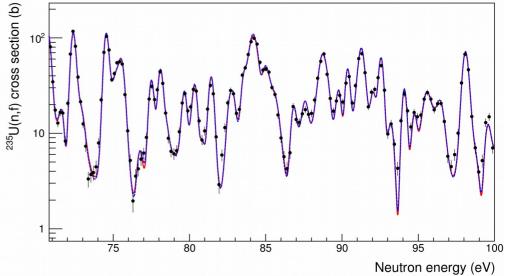
²³⁵U(n,f) at lower energies



High quality data also at lower energies. A resonance analysis will be done soon.











Summary

- An accurate measurement of ²³⁵U(n,f) cross section with respect to ⁶Li(n,t) and ¹⁰B(n,α) standard has been performed at n_TOF between thermal and 170 keV neutron energy. Paper accepted for publication on EPJ-A!
- An overestimation of the fission cross-section in libraries has been revealed in the 9-18 keV neutron energy range.
- ◆ The measurement also evidences the presence of structures in the 2.2 4 keV range, just after the end of RRR of ENDF8 and JEFF3.3.
- ◆ High quality data have been collected at lower energies and a detailed resonance analysis will be performed.
- ◆ Silicon detectors are well suitable for high precision fission measurement at n_TOF.



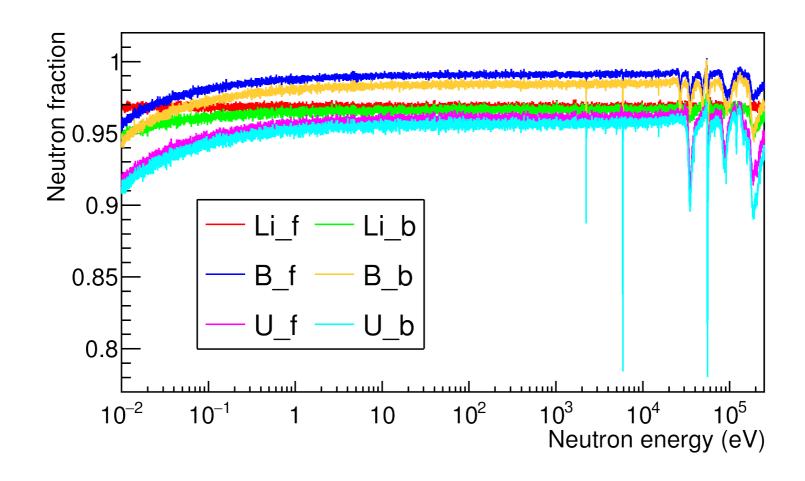


Thank you for your attention





Absorption correction

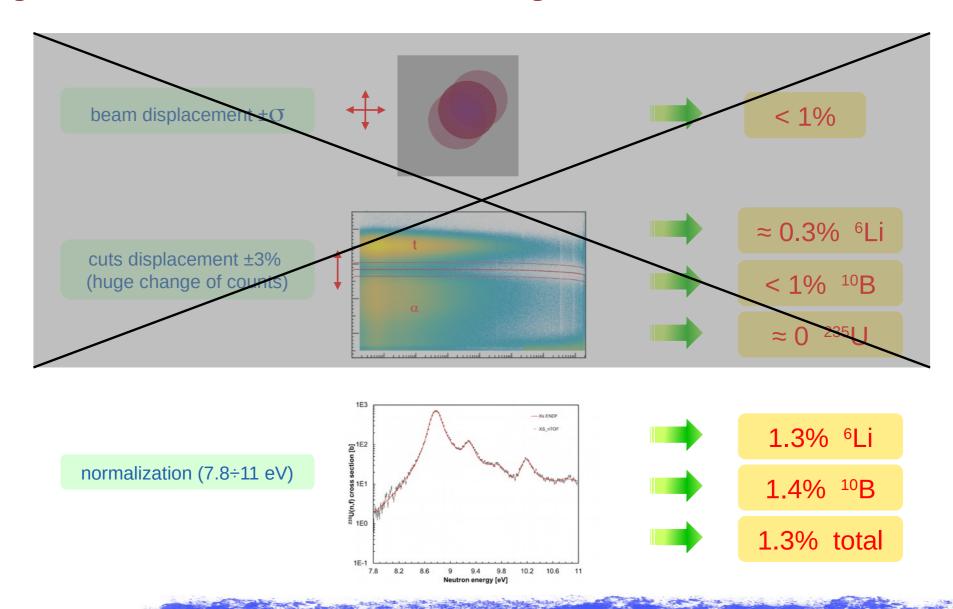








Systematic uncertainty

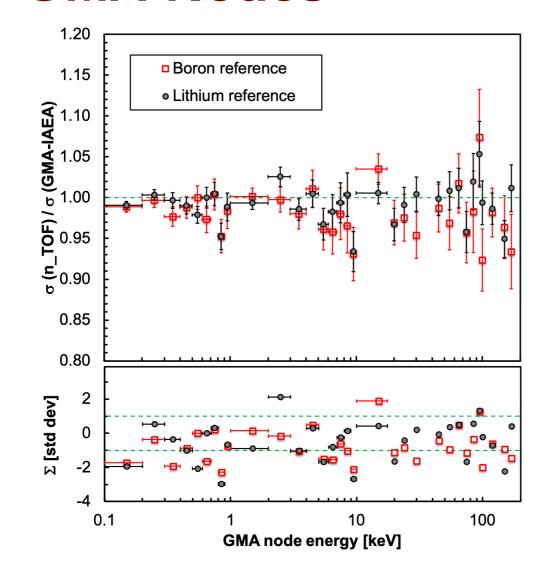








GMA-Nodes



Overall agreement with point-wise reference provided by IAEA in 2018^[1].

Differences for GMA points at 0.85 and 9.5 keV.

[1] Carlson et al., "Evaluation of the Neutron Data Standards", Nuclear Data Sheets 148 (2018) 177



